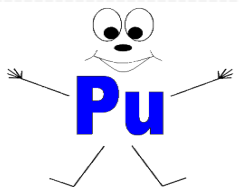




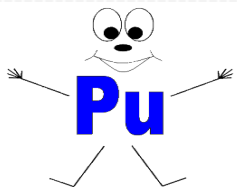
RAPID METHOD FOR DETERMINATION OF ACTINIDES AND Sr FROM SMALL AMOUNTS OF SAMPLES

Edit Bokori, Nóra Vajda, Zsuzsa Molnár



CONTENT

- Long-lived radionuclides from nuclear facilities
- Rapid separation procedure for actinides
 - Examples for actinides measurements
- Extension of rapid method for Sr isotopes determination by β spectrometry
 - Examples for Sr determination



ACTINIDES

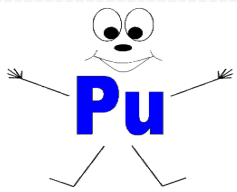
LONG-LIVED ACTINIDES RELEVANT IN ENVIRONMENTAL MONITORING AND RADIOACTIVE WASTE ANALYSIS

Isotope	Half life	Decay mode	Specific activity Bq/g
⁸⁹ Sr	50 d	β ⁻	1,09E+15
¹³⁴ Cs	2,06 y	β ⁻ , γ	4,78E+13
⁵⁵ Fe	2,73 y	EC	8,36E+13
¹²⁵ Sb	2,76 y	β ⁻ , γ	3,82E+13
⁶⁰ Co	5,27 y	β ⁻ , γ	5,88E+13
²²⁸ Ra	5,75 y	β ⁻	1,01E+13
³ H	12,30 y	β ⁻	3,57E+14
²⁴¹ Pu	14,35 y	β ⁻	3,82E+12
^{93m} Nb	16,1 y	EC	8,81E+12
²⁴⁴ Cm	18,1 y	α	3,00E+12
²¹⁰ Pb	22,3 y	β ⁻ , γ	2,83E+12
⁹⁰ Sr	29,1 y	β ⁻	5,06E+12
¹³⁷ Cs	30,2 y	β ⁻ , γ	3,20E+12
²³⁸ Pu	87,7 y	α	6,34E+11
⁶³ Ni	100,1 y	β ⁻	2,10E+12
²⁴¹ Am	432,2 y	α	1,27E+11
²²⁶ Ra	1600 y	α	3,66E+10
¹⁴ C	5730 y	β ⁻	1,65E+11
²⁴⁰ Pu	6563 y	α	8,40E+09
⁹⁴ Nb	20300 y	β ⁻ , γ	6,91E+09

Isotope	Half life	Decay mode	Specific activity Bq/g
²³⁹ Pu	24110 y	α	2,30E+09
²³⁰ Th	75380 y	α	7,63E+08
⁵⁹ Ni	76400 y	EC, β ⁺	2,94E+09
⁴¹ Ca	0,103 My	EC	3,14E+09
²³³ U	0,1492 My	α	3,80E+08
⁹⁹ Tc	0,211 My	β ⁻	6,34E+08
²³⁴ U	0,2455 My	α	2,30E+08
³⁶ Cl	0,301 My	β ⁻	1,22E+09
²⁴² Pu	0,3733 My	α	1,46E+08
⁷⁹ Se	1,13 My	β ⁻	2,08E+08
⁹³ Zr	1,53 My	β ⁻	9,27E+07
²³⁷ Np	2,144 My	α	2,60E+07
¹³⁵ Cs	2,3 My	β ⁻	4,26E+07
¹⁰⁷ Pd	6,5 My	β ⁻	1,90E+07
¹²⁹ I	15,7 My	β ⁻	6,49E+06
²³⁶ U	23,4 My	α	2,40E+06
²⁴⁴ Pu	80,8 My	α	6,71E+05
²³⁵ U	703,8 My	α	8,00E+04
²³⁸ U	4468 My	α	1,24E+04
²³² Th	14050 My	α	4,06E+03

Analyses:
by alpha and beta spectrometry

Requirements: complete separation



RAPID METHOD

Recommended procedure IAEA/AQ/11 (2009) developed by Nóra Vajda, et al
Simultaneous determination of actinides on a single column filled **with TRU resin**
from small amounts of sample

Advantage for emergency and nuclear waste analysis

rapid: within 1 day
small sample: 0.1 – 1 g solid sample
less than 10 ml liquid sample

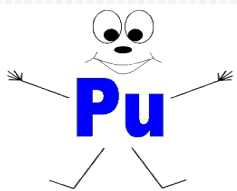
Provide adequate information on environment in emergency situation

Characterisation of nuclear waste

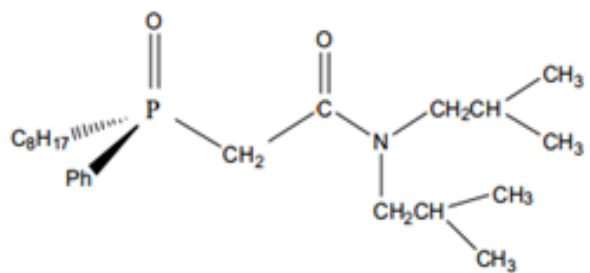
Control of contamination after work with radioisotopes (Decontamination and

Decommissioning of nuclear facilities)

Safeguards



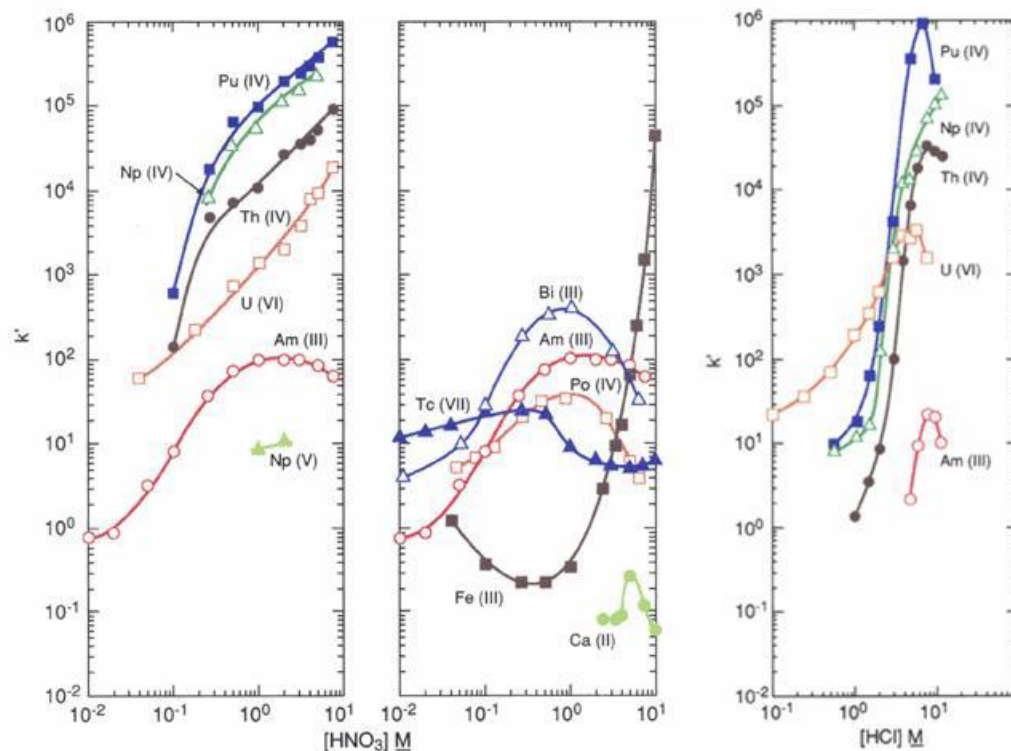
TRU resin



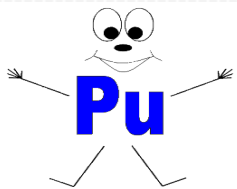
carbamoyl-methyl-phosphine oxide (CMPO)

- Extractant: CMPO / TBP
- Retention of Pu(IV) Th(IV) Np(IV)
- U(VI) and Am(III)

Acid dependency of k' for various ions at 23-25°C.
TRU Resin



Horwitz, et al. (HP193)



FLOW CHART OF RAPID RADIOCHEMICAL PROCEDURE

ACID DIGESTION or FUSION
dissolution: 1M HCl



Fast and complete destruction

ADDITION TRACERS & CARRIER
Tracer: ^{242}Pu , ^{243}Am , ^{232}U
Carrier : Sr



to determine chemical recovery

reduction
 CaF_2 or $\text{Ca}(\text{COO})_2$
CO-PRECIPIATION

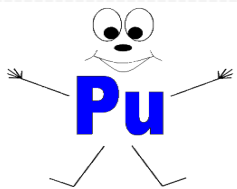
CHEMICAL SEPARATION
TRU extraction chromatography

SOURCE PREPARATION
micro-coprecipitation

MEASUREMENT
by α spectrometry

Advantage

less reagents and chemicals are used
less hazardous waste is produced, and
the process is more economic



Complete sample destruction /dissolution

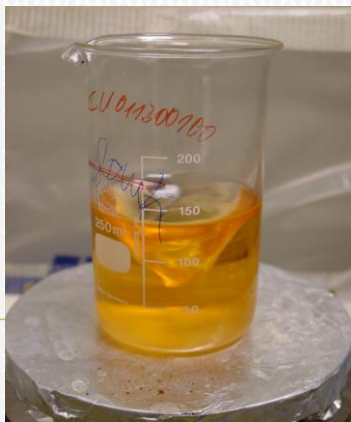
Samples: **liquid** or **solid** environmental samples or waste from NPP

Conventional technique

wet digestion: evaporation on a hot plate with mineral acids

to destroy organics, possible complexing agents

<10 ml liquid waste



Rapid technique

Fusion: In Pt crucible with gas burner with alkaline flux

The fusion melt is dissolved in 1M HCl up to 1 g of soil or sediment

0.5 g dry sample

Reagents

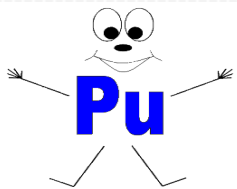
0.2 g NaNO_3 (oxidizing agent)

0.2 g Na_2CO_3 (additive)

2.0 g LiBO_2

0.1 g KI (non wetting agent)





Co-precipitation of actinides

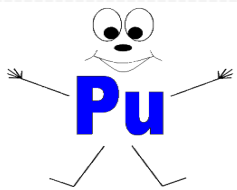
necessary reductive media to avoid losses: Pu(III) U(IV)
+Mohr's Salt + hydrazine

CaF₂

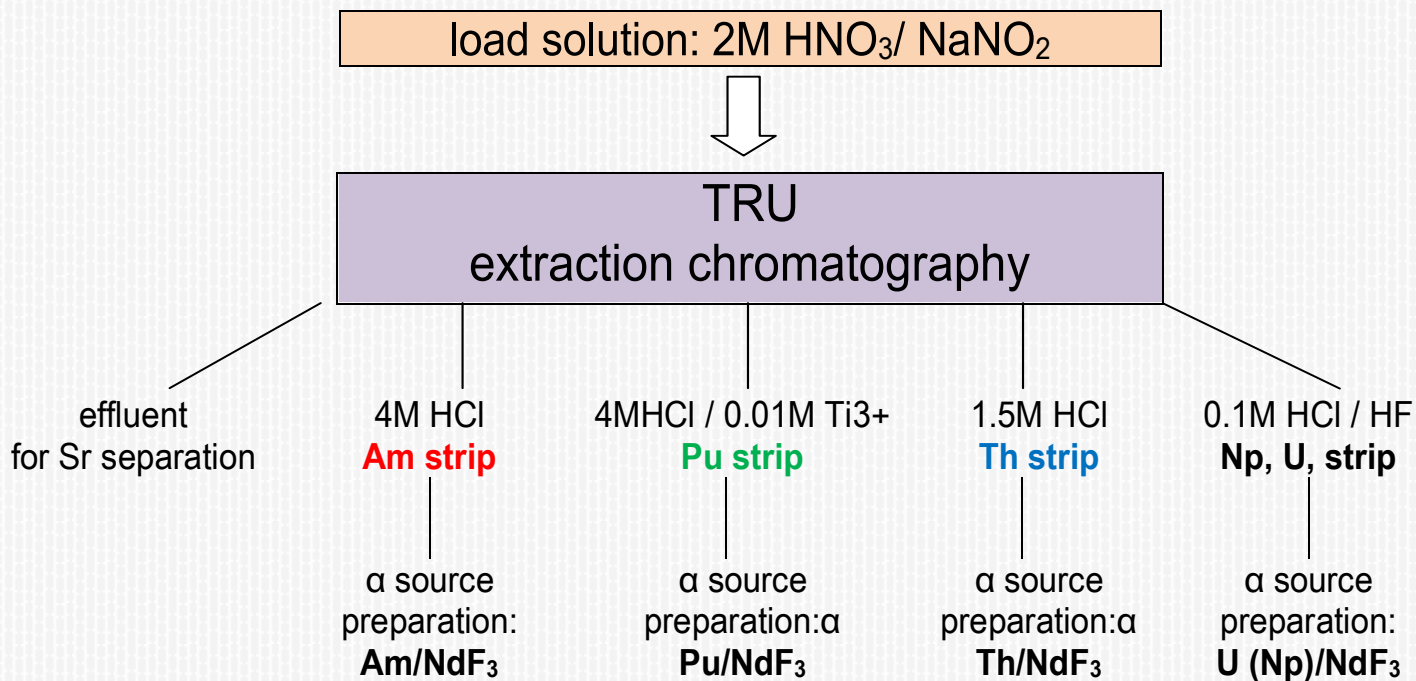
- fusion melt is in 100 ml 1M HCl
 - + Ca
 - + 20-30 mL 40% HF
- Heated then cooled down
- Filtration through 0.45 μm pore size membrane.
- Solution in 2M HNO₃ / H₃BO₃

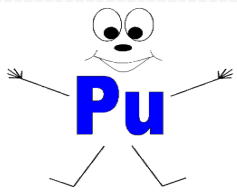
Ca (COO)₂

- Dry residue is solved in 50 ml 1M HCl
- Ca and oxalic acid to samples
 - pH 4-5
 - heated , cooled down
- Filtration through 0.45 μm pore size membrane
- destroy oxalate with cc HNO₃
- dry residue is taken up in 2 M HNO₃
- adjust oxidation state with hydrazine and stabilize with NaNO₂



Extraction chromatography





Source preparation and nuclear measurement

α Source preparation

Strip solution

50 μ g Nd/100 μ l

5 ml 40 % HF

Pu , U +Mohr's salt

Filtration of micro-precipitate

Measurement by α spectrometry

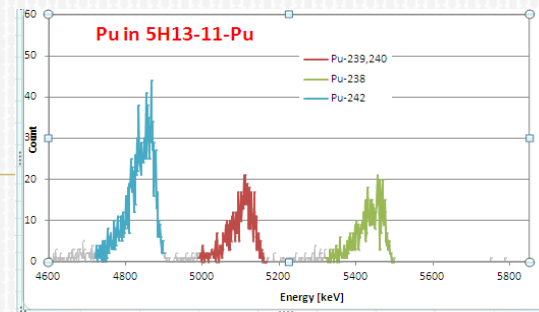
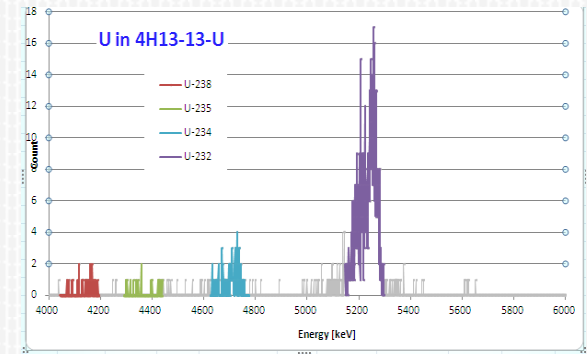
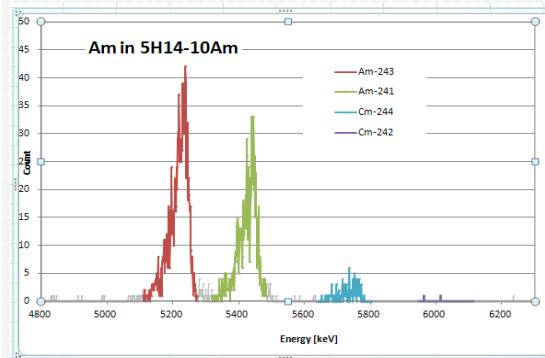
Si semiconductor (PIPS) detector

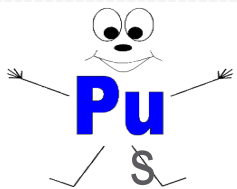
Detection limit 0.1 mBq

Requirements for α source

Purity of source

Infinitely thin layer \rightarrow good resolution of α spectra





Chemical yield

radioactive waste routine analysis

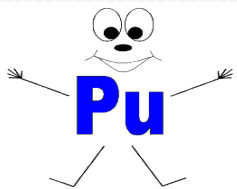
Waste samples from NPP 2012

Sample	U yiel	Pu yield	Am-Cm- yield
V mL	%	%	%
H12-9	0.0015	41%	65%
H12-11	1	34%	67%
H12-12	0.01	69%	46%
H12-13	10	69%	75%
H12-14	10	87%	56%
H12-15	1	94%	48%
H12-4	10	70%	119%
H12-5	10	81%	95%
H12-6	10	92%	100%
Average		71%	75%

Environmental reference material analysis

Soil and sediment samples

Sample	Am yield	Pu yield	Th yield	U yield
code				
IAEA-384	96%	91%	72%	91%
IAEA-367	100%	92%	95%	92%
IAEA-368	93%	94%	73%	81%
NIST-4357	103%	91%	94%	91%
IAEA-385	91%	59%	84%	21%
IAEA-135	89%	75%	83%	43%
Average	96%	85%	84%	75%

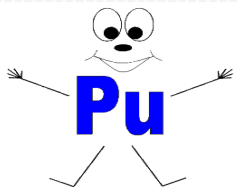


Results of analysis of waste samples from NPP using TRU resin

Container code	top layer		middle layer		bottom layer	
	act. (Bq/l)	σ (Bq/l)	act. (Bq/l)	σ (Bq/l)	act. (Bq/l)	σ (Bq/l)
U-234	3,68E+00	\pm 9,0E-01	4,34E+00	\pm 9,9E-01	3,39E+01	\pm 6,9E+00
U-235		\leq 1,2E+00		\leq 1,5E+00		\leq 7,7E+00
U-238	1,61E+00	\pm 5,6E-01	8,60E-01	\pm 4,9E-01	8,79E+00	\pm 3,6E+00
Pu-239,240	1,21E+02	\pm 5,8E+00	1,53E+02	\pm 5,4E+00	1,25E+04	\pm 4,1E+02
Pu-238	1,35E+02	\pm 6,3E+00	1,88E+02	\pm 6,6E+00	1,38E+04	\pm 4,5E+02
Am-241	5,05E+01	\pm 6,0E+00	1,00E+02	\pm 4,7E+00	1,33E+04	\pm 4,6E+02
Cm-244	1,29E+01	\pm 2,8E+00	1,48E+01	\pm 1,4E+00	2,77E+03	\pm 1,2E+02
Cm-242		\leq 6,6E+00		\leq 1,4E+00	3,81E+01	\pm 9,5E+00

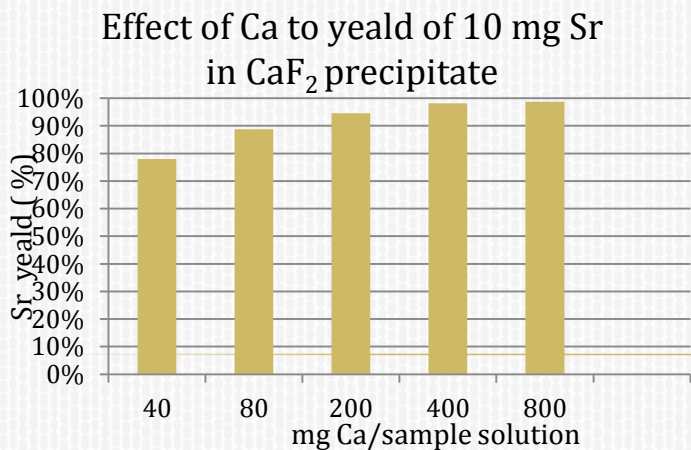
≤ 10 mL
sample

TRU
method



Sr separation fitted to rapid method for actinide

ICP MS measurements	Chemical yields (%) during CaF ₂ co precipitation			
	IAEA soil-6	IAEA- SL-3 sediment	NBS- 4355	NIST694 phosphate ore
Ca	46	34	17	45
Sr	48	74	83	67



ACID DIGESTION or FUSION
dissolution: 1M HCl

ADDITION TRACERS & CARRIER
Tracer: ²⁴²Pu, ²⁴³Am, ²³²U
Carrier : Sr

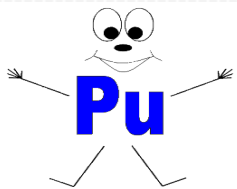
reduction
CaF₂ or Ca(COO)₂
CO-RECIPIATION

CHEMICAL SEPARATION
TRU extraction
chromatography

Sr separation on Sr resin from effluent of TRU column

SOURCE PREPERATION
micro-coprecipitation

MEASUREMENT
by α spectrometry



Method for Sr determination

a

Sr separation

effluent from TRU column (2M HNO_3)



Adjust load to 3M HNO_3



Radiochemical separation of Sr
3 g Sr resin in column



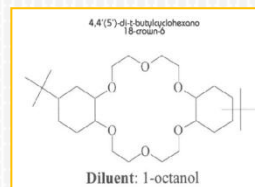
Sr-oxalate precipitation



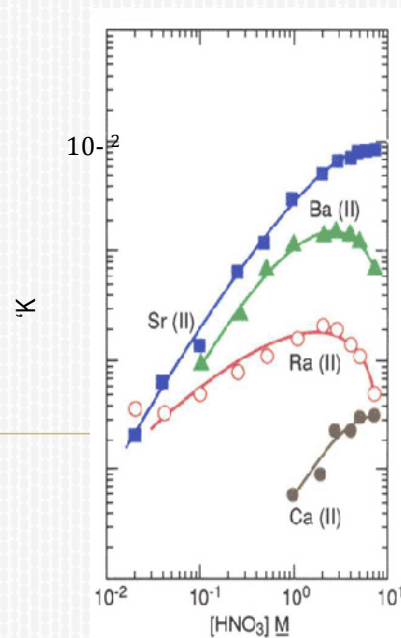
Measurement of Sr isotopes
by LSC

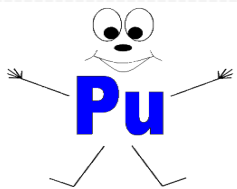


Chemical yield
determination by
gravimetry



acid dependence of
 k' for alkaline earth
element





Results of analysis of Sr isotope in swipe and waste samples

RA cod	DM1		DM2	
	Calcium oxalate precipitate			
isotopes	yeald %	activity± σ Bq/sample	yeald %	activity± σ Bq/sample
Sr-89		LD ≤ 0.01		LD ≤ 0.009
Sr-90	64%	0.477 ± 0.058	63%	0.37 ± 0.058

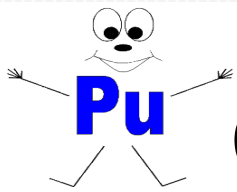
RA cod	H12-12			H12-11		
	Calcium fluorid precipitate					
	ml	yeald	activity± σ Bq/l	ml	yeald	activity± σ Bq/l
Sr-89			LD ≤ 6.8E+03			LD ≤ 2.6E+02
Sr-90	0.1	86%	3.04E+05 ± 2.3E+04	1	79%	3.5E+03 ± 2.5E+02

actinide recovery from swipe samples

	chemical yield	
	DM1	DM2
	oxalate pr.	
U	89%	95%
Pu	60%	52%
Am	59%	60%

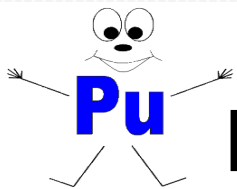
actinide recovery from waste samples

	chemical yield	
	H12-12	H12-11
	fluorid pr.	
U	63%	54%
Pu	66%	76%
Am	72%	67%



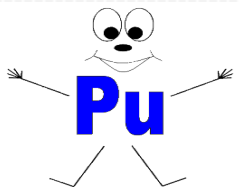
Conclusion

- This radiochemical procedure is capable for rapid determination of actinides (Am, Pu, Th, and U radioisotopes) and Sr isotopes in small amounts of contaminated samples (swipe, waste, sediment) by alpha spectrometry.
- The procedure is based on wet digestion of liquid samples and fusion of solid samples with LiBO_2 .
- CaF_2 or $\text{Ca}(\text{COO})_2$ co-precipitation is used for pre-concentration of actinides and Sr.
- Ca addition to the sample makes Sr precipitation complete and the method becomes suitable for Sr isotopes determination from the same samples.



References

- N. Vajda, C. K. Kim: Determination of Pu isotopes by alpha spectrometry: a review of analytical methodology, J. Radioanal. Nucl. Chem. 2010, 283, 203-223
- N. Vajda, C. K. Kim: Determination of ^{241}Am by alpha spectrometry: a review of analytical methodology, J. Radioanal. Nucl. Chem., 284(2), 341-366, 2010.
- N. Vajda, A. Törvényi, G. Kis-Benedek, C.K. Kim, B. Bene, Zs. Macsik: Rapid method for the determination of actinides in soil and sediment samples by alpha spectrometry. Radiochimica Acta 2009, 97, 395-401.



Thank you for your attention

