

EXTRACTION CHROMATOGRAPHY

Technical Documentation- Radiopharmacy



Extraction Chromatographic Resins

Specialty Ion Exchange Resins

Novel Separation Materials

Method Development

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TRISKEM INTERNATIONAL



Triskem International develops, manufactures and commercializes highly selective resins for the separation of a large variety of elements in numerous domains.

Triskem Resins are used for:

- the analysis of precious samples (environmental monitoring, radiation protection, bioassay)
- the recuperation of high value elements (production of radio metals for medical use and recuperation of their respective target materials and recovery of critical metals)
- the high precision determination of isotope ratios (dating/geochronology/provenancing)
- the removal of particular elements before effluent release
- quality control of final and intermediary products in numerous industries including radiopharmacy

We also commercialize a complementary **range of accessories** such as vacuum box systems and products used for sample preparation like LSC vials, filters and discs for alpha spectrometry.

Our technical support is carried out by a **worldwide recognized scientific team** consisting of radiochemistry PhDs who are amongst others members of the French nuclear equipment standardization agency (BNEN), ISO workgroups, and external IAEA experts and lecturers.

Triskem International products have now become **the standard technology** in the field of radiochemistry, geochemistry and radiopharmacy with methods used by accredited laboratories (Orano, BNFL, CEA, NNL,...) and regulatory organizations (IRSN, IAEA, BRGM,...) and many other customers working on radionuclide and element separation and purification.

APPLICATIONS

Radiopharmacy

TrisKem manufactures selective resins for Lanthanides (e.g. Lu-177, Tb-161,...), Cu, Zr, Ga, Ac, Ge, Ti, Sc, Tc and many other radionuclides for medical purposes. We develop rapid, highly specific separation techniques for:

- Separation of radionuclides from irradiated targets
- Quality control of radionuclides for medical use
- Post-generator purification
- Radioprotection and Radioanalysis

Environmental Monitoring and Bioassay

TrisKem's extraction chromatographic resins allow the separation and determination of radionuclides (e.g. actinides and fission products) from bioassay samples and environmental matrices e.g. **soil, sediment, vegetation and seawater samples**. Their use allows a rapid and precise determination of radionuclides in emergency and routine situations, as well as, combined with suitable sample preparation methods, to obtain very low detection limits through analysis of large volume samples.

Decommissioning

The high selectivity of our resins not only allows the analysis of standard alpha and beta emitters (actinides, Sr-89/90, Ni-63, Tc-99,...) from high matrix samples including concrete, spent resin, but also the determination of so called difficult-to-measure (DTM) radionuclides such as **Sn-121m/Sn-126, Zr-93**,...

We are constantly working on developing new methods according to your needs. Methods for the following radionuclides are currently under development: **Se-79, Sb-125, Cs-135, Nb-93m/Nb-94, Ag-108m, Pd-107**,...

Geochemistry and Metals Separation

TrisKem's expertise in separations and its variety of different resins with varying selectivities provides a number of opportunities to solve analytical problems through sample preparation. With our broad offer of products and long experience, TrisKem can help you meet lower detection limits with less uncertainty. The problems encountered are frequently centered on removing matrix that interferes with the instrumental measurement. ICP-MS is an example. Isobaric interferences often have to be removed in order to allow accurate determination of your analytes. Selectivity for the analyte of interest is important in these cases. Extraction chromatography is already widely used in various applications including **geochronology, isotope ratio determination and provenancing**.



A number of ready-to-use methods are available through our new website: www.triskem.com

CL Resin

The CL Resin is based on an extraction system that is selective for soft cations such as palladium, gold and silver and it is mainly used for the separation of chloride and iodide, especially in the context of Cl-36 and I-129 analysis, the separation of iodine isotopes and the removal of radioiodine from effluents.

The selectivity for halides is introduced by loading the resin with Ag^+ allowing good selectivity for anions, especially halides, forming sparsely soluble or insoluble Ag complexes. Since the resin retains Ag^+ over a wide range of pH values it also allows loading chloride and iodide from various conditions, from slightly (chloride) to highly alkaline (iodide) to strongly acidic (both), ideally under reducing conditions to ensure their presence as halides.

Radioiodine is considered to be one of the most dangerous radioelements in terms of radiological effects in case of accidental release. Accordingly efforts are made to capture iodine from liquid effluents already during the production process before waste storage, to avoid a possible gas release from stored liquid waste at a later date.

Caroline Decamp of the Intitute of Radioelements (IRE) has developed a method for the removal of radioiodine from acidic (1M HNO_3) radioactive process waste solutions. The IRE combined the CL Resin with XAD-4 resin and packed this mix (4g XAD-4 and 3g CL Resin) into columns. These mixed bed columns were then introduced into their process setup in order to remove radioiodine from their multi-curies production process solutions.

One prerequisite of the removal step was that it should not slow down the process; the radioiodine removal was thus tested at the same flowrates applied during the process. Flow rates up to $>180\text{ mL/min}$ and effluent volumes between 12L and 17L were tested, and it was found that under these conditions between 85 and more than 95% (overall mean retention yield for all solution volumes is $88\% \pm 5\%$ ($N = 14$, $k = 1$)) of the radioiodine present were retained on the column, thus lowering the I-131 activity of the effluents by a factor of 10 from 100 GBq/L to about 10 GBq/L . 2000 GBq of I-131 could be removed per decontamination cycle and stored as solid waste.

Mastren et al. used the CL Resin for the separation of Ag-111 from a proton irradiated thorium target.

In a second paper they have further shown that the CL Resin may be used to separate Pa from a bulk thorium matrix.

Besides the removal of radioiodine from effluents the CL Resin is also used for analytical purposes. It allows for the preconcentration of I isotopes e.g. from waste water upfront to gamma spectrometry and, more importantly, for the determination of I-129 and Cl-36, both long-lived, volatile radionuclides, in environmental and decommissioning samples.

In case of analytical applications, after loading and rinsing of the resin (removal of matrix elements and interferents), chloride can be easily eluted from the resin using dilute SCN^- solutions whereas iodide remains fixed. Iodide can then be eluted from the resin using a moderately concentrated S^{2-} solution.

Main applications

- Separation and removal of radioiodine
- Ag separation
- Pa separation
- Cl-36 and I-129 separation



CU Resin

The CU Resin is used for the separation of Cu and is based on a Cu selective extraction system.

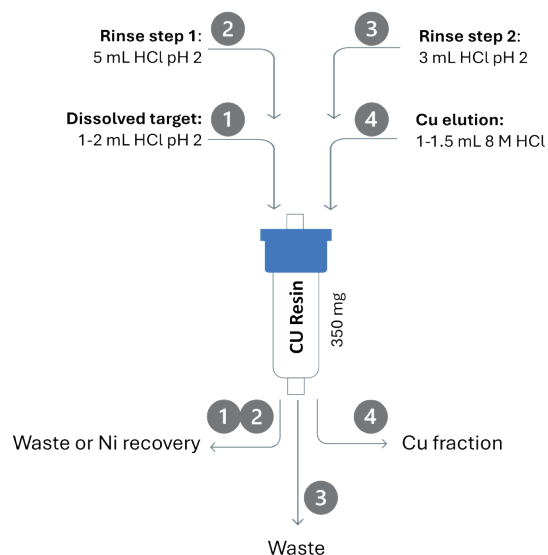
The extraction behaviour of the CU Resin towards a number of elements in three different acids at varying acid concentrations is presented here.

Overall the resin shows high selectivity for Cu over all tested cations including Ni and Zn. Cu uptake is generally high at pH values greater than 2 while it can be easily eluted with mineral acids of elevated concentrations. Accordingly it is well suited for liquid target work. For Cu separation from solid Ni targets on the other hand the TK201 Resin is more suitable.

Further the resin shows high robustness against interference of elevated amounts of Zn and Ni, even at 1g of Zn or Ni per g of CU Resin employed the $D_w(\text{Cu})$ remains greater than 1000.

A simple and fast method for the separation of Cu from irradiated targets was developed by Dirks. et al. allowing to obtain highly pure Cu in a very small volume in less than 10min.

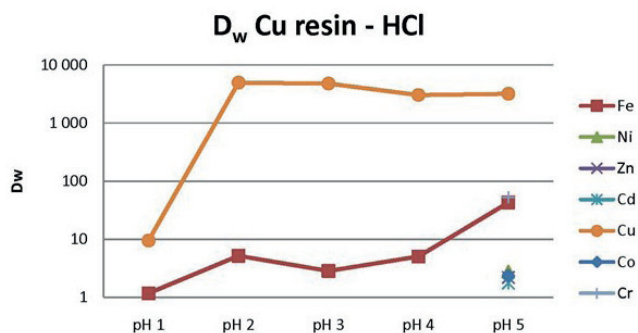
DeGraffenreid et al. could show that the CU Resin allows separating Cu from very large Zn targets in high yields and purity. It may be combined with the TK201 Resin to convert the obtained Cu fraction from high acid concentration to dilute acid.



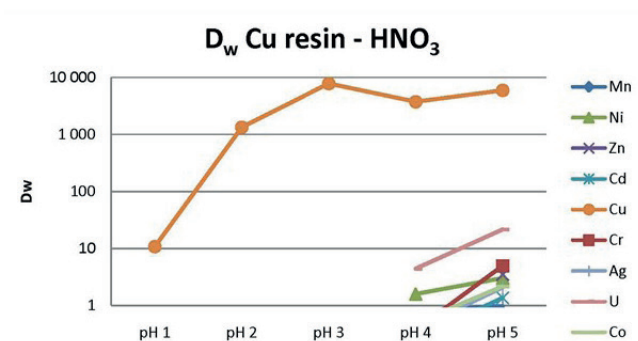
Optimized Cu separation method

Thieme et al. showed that removing traces of stable Cu from Ni targets will result in a higher specific activity of Cu isotopes produced from such targets.

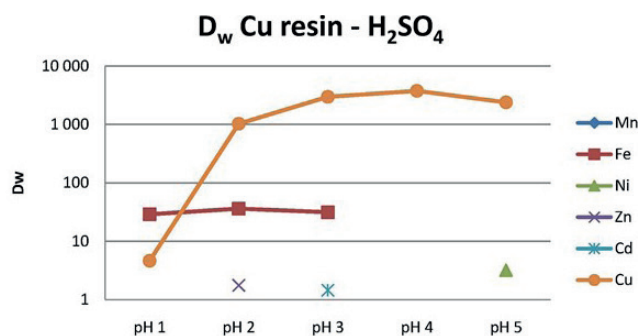
It could further be shown that Cu can be concentrated, and separated, from other high-matrix samples such as sea water, biological and mining samples with high yield and purity.



D_w of Cu and selected elements on CU Resin in HCl at varying pH values.



D_w of Cu and selected elements on CU Resin in HNO₃ at varying pH values.



D_w of Cu and selected elements on CU Resin at H₂SO₄ in varying pH values.

Main applications

- Separation of Cu-64/67 from irradiated targets
- Concentration and separation of Cu from environmental matrices
- Purification of target materials

CU iSheets

CU iSheets are comprised of iTLC-SG TLC (Thin Layer Chromatography) paper (Agilent) impregnated with the same Cu selective extractant that is also employed in the CU Resin.

TLC papers are frequently used in the quality control (determination of radiochemical purity) of labeled compounds for use in radiopharmacy. In some cases, for example when analyzing Cu-labelled peptides, artefacts can form during such a TLC test when using silica gel based supports. These artefacts can then interfere with the analysis of TLC scans by creating or distorting peaks. While switching to non-silica based TLC supports (e.g. Whatman paper) generally leads to an improvement, this comes with the disadvantage of significantly longer development times and broad peaks.

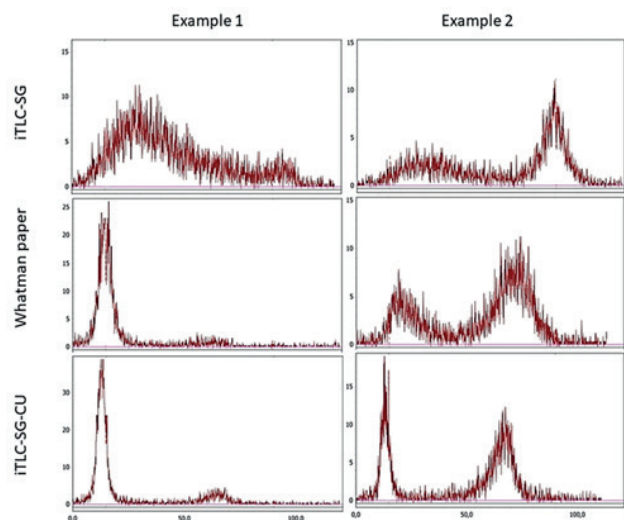
Svedjehed and Gagnon could show that using CU iSheets a significantly better resolution with short development times can be obtained for Cu labelled peptides.

The authors produced ^{61}Cu via the $^{\text{nat}}\text{Ni}(\text{d},\text{x})^{61}\text{Cu}$ reaction using a GE PETtrace solid target system and purified the $^{61}\text{Cu}\text{CuCl}_2$ using a TBP/TK201 Resin based method as described by Gagnon et al.

Aliquots of $^{61}\text{Cu}\text{CuCl}_2$ were then incubated (90°C; 30min; pH 4.4 [0.3M acetate buffer]) with varying low concentrations of NOTA-octreotide trifluoroacetate or NODAGA-RGD trifluoroacetate. In all cases the ligand concentration was kept below excess to ensure incomplete labelling and thus presence of non-labeled ^{61}Cu .

5 μL of the respective ^{61}Cu -labelled peptide were then spotted with origin at 1cm onto strips of 10cm length of: a.) iTLC-SG, b.) Whatman paper (both non-impregnated) and c.) CU iSheets, and developed to at least 7cm in 1:1 MeOH/1M ammonium acetate.

The following fig. shows the result of TLC scans of $^{61}\text{Cu}\text{Cu-NOTA-octreotide}$ with elevated levels of free ^{61}Cu (example 1) and $^{61}\text{Cu}\text{Cu-NOTA-octreotide}$ scans at comparable levels of labelled and unlabelled ^{61}Cu (example 2). The authors reported that similar results were obtained in case of $^{61}\text{Cu}\text{Cu-NODAGA-RGD}$.



TLC scans of $^{61}\text{Cu}\text{Cu-NOTA-octreotide}$ spotted on:
top: iTLC-SG; middle: Whatman paper; bottom: CU iSheets.
Example 1 notes elevated levels of unlabelled ^{61}Cu , while example 2 notes comparable levels of labelled to unlabelled ^{61}Cu (data provided by Svedjehed et al.)

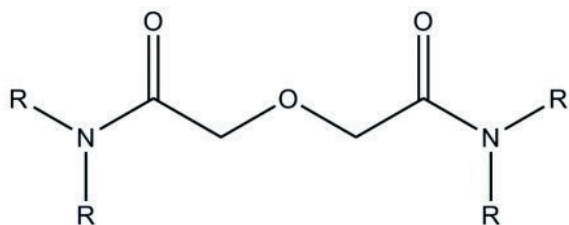
The scans show that in this case using non-impregnated iTLC-SG and Whatman papers results in wide, non-uniform peaks of low resolution. TLC scans using CU iSheets show significantly sharper, separated peaks with the added benefit of short development time compared to the Whatman paper (<10min instead of ~25-30min for the Whatman paper). The authors report even greater differences when decreasing the amount of peptide.

Main applications

- Quality control of Cu labelled compounds via TLC



DGA Sheets

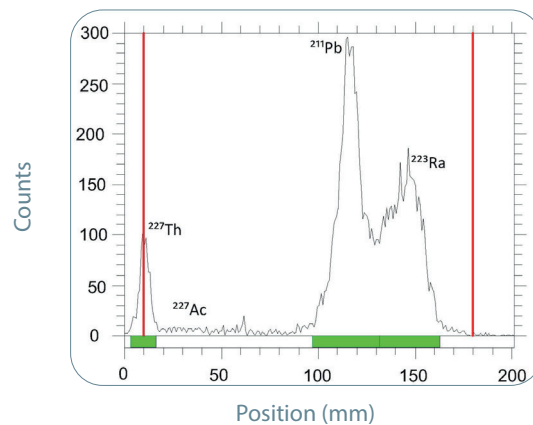


Radionuclide separation and radionuclidic purity determination was never easier. Separation of generator isotopes and radionuclide mixtures on DGA impregnated chromatographic paper, which has been developed at the CVUT, including mixtures like Ac-227/Th-227/Ra-223, Sr-90/Y-90, Ge-68/Ga-68, Mo-99/Tc-99m, Pb-212 and Ac-225/Bi-223 is now possible using one separation material, just by changing the composition of the mobile phase (diluted mineral acids like 1M HNO₃ or HCl). The chromatographic paper is impregnated with DGA with variable active compound loading (0.1-10%).

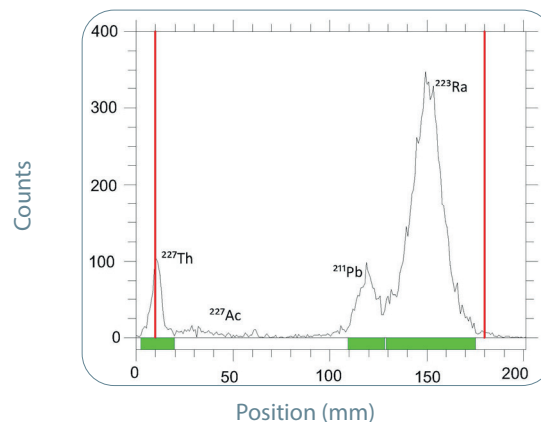
The method is easy to validate and TLC scanners or alternatively, after cutting the paper, common radiometers may be used to determine radionuclidic purity of a generator eluate or a purified radionuclide product.

Proposed standard sheet dimensions are 5 x 20cm, 10 x 10cm or 20 x 20cm. Other formats and custom dimensions are however also available upon request.

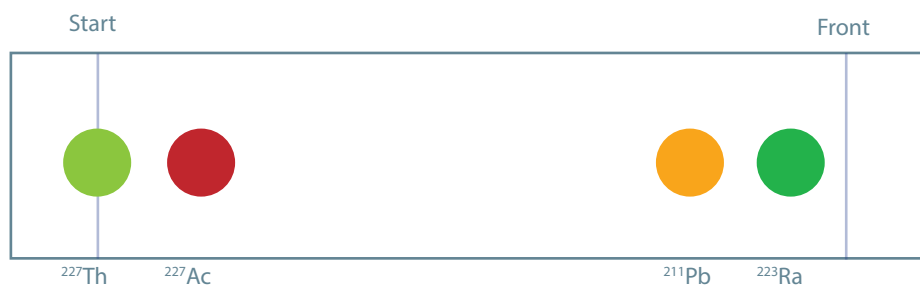
Soon also available as impregnated iTLC paper version.



Radiochromatogram measured immediately after separation



Radiochromatogram measured one hour after separation



Scheme of a chromatographic separation of a mixture of Ac-227 and its descendents. Th-227 remains on start, Ac-227 has a retention factor R_f of ca. 0.2, Pb-212 of ca. 0.7 and Ra-223 of ca. 0.9.



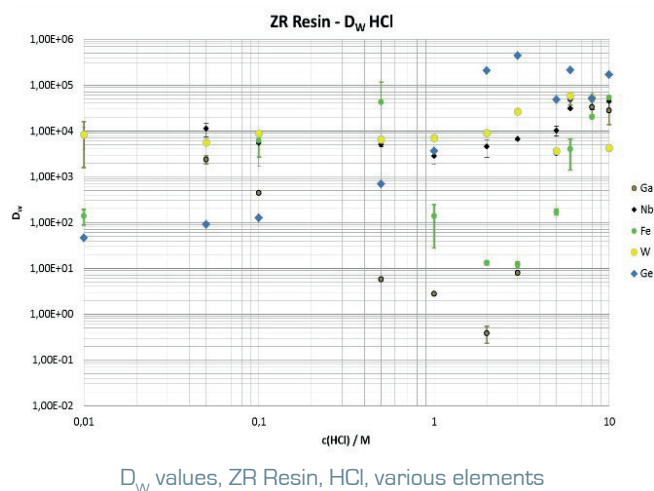
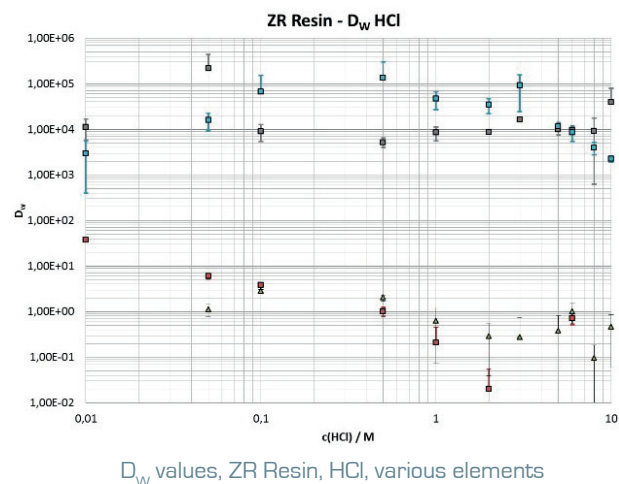
All data provided by J. Kozempel and M. Vlk, CVUT

Application notes available on our website: www.triskem.com/tki-methods.php

ZR Resin

The ZR Resin is based on the hydroxamate functionality frequently used for the separation of zirconium, especially from Y target materials, for later use in radiopharmaceutical applications.

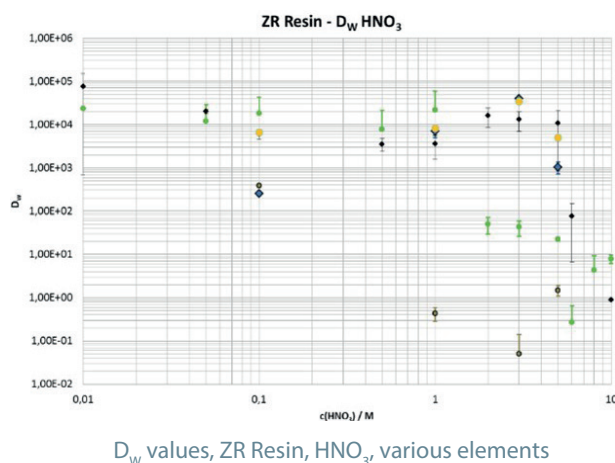
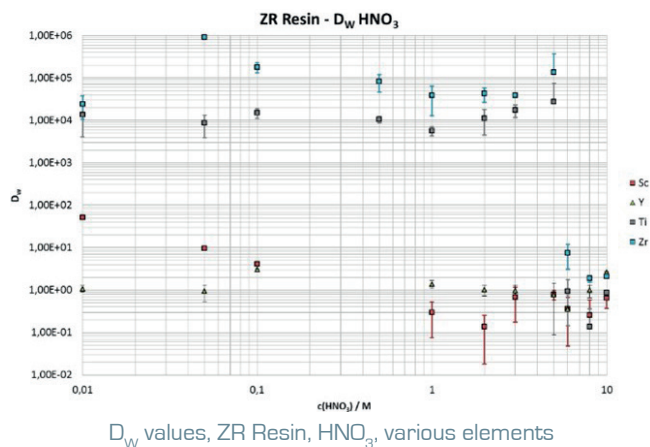
Dirks et al. characterized the resin with respect to its selectivity for selected elements in HNO_3 , HCl and oxalic acid; results are summarized in the following figures.



The ZR Resin shows high selectivity for Zr, Ti and Nb over a wide range of HCl concentrations (0.01M – 10M), Fe(III) is strongly retained at low and elevated HCl concentrations, retention is weak from 1 – 6M HCl . As expected, the resin shows very little selectivity for Sc and Y, a separation e.g. of Zr from Y and of Ti from Sc seems thus feasible.

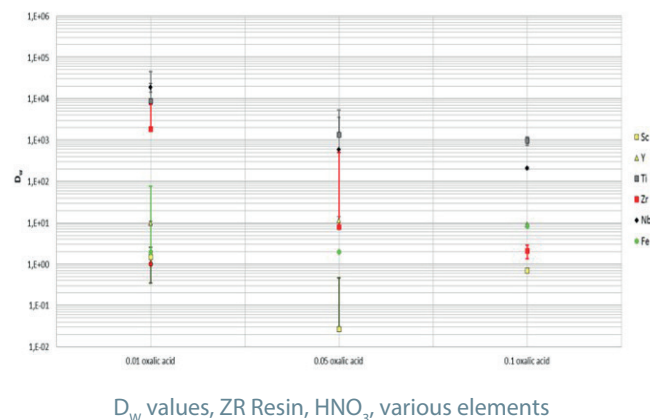
The resin further shows quite interesting selectivity with respect to Ga and Ge.

Ga is very well retained at low HCl concentrations ($\leq 0.1\text{M}$) as well as at high concentrations ($\geq 5\text{M}$ HCl), while Zn e.g. is not retained at all. At HCl concentrations in-between, particularly at 1-2M HCl it is not retained. Ge on the other hand is very well retained at HCl concentrations $> 0.1\text{M}$. Especially at 2M HCl the selectivity for Ge over Ga is very high.



The resin shows rather similar selectivity in HNO_3 . Zr, Ti and Nb are well retained up to 5M HNO_3 , Fe(III) is well retained up to 1M HNO_3 . At higher HNO_3 concentrations the nitric acid starts decomposing the extractant, as indicated by a colour change of the resin from white to brown; accordingly, the resin shows no significant selectivity towards the tested cations under these conditions. As in HCl , Y and Sc show no significant retention on the ZR Resin in HNO_3 .

It should be noted that the ZR Resin also shows high selectivity for Ge over Ga (and Ni/Co) at elevated HNO_3 concentrations.



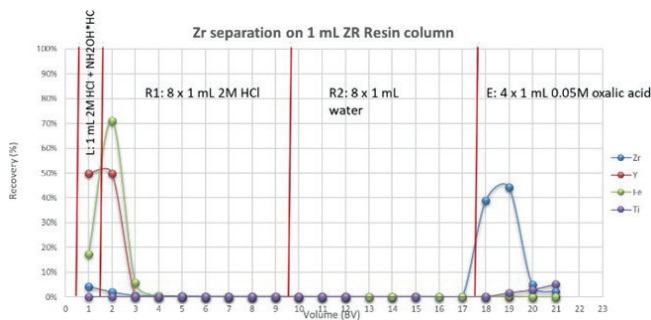
Oxalates are very strong complexing agents for Zr, accordingly they are very frequently used for the elution of Zr.

It could be shown that oxalic acid concentrations above 0.05M lower the D_w value of Zr on the ZR Resin strongly; they are thus suitable eluting agents for Zr. It was further observed that Nb shows rather elevated D_w values even at 0.05M oxalic acid, indicating that Zr and Nb may be separated by adjusting the oxalic acid concentration accordingly.

Based on obtained D_w values several elution studies were performed with main focus on the use of the resin in the context of radionuclide production for radiopharmaceutical use.

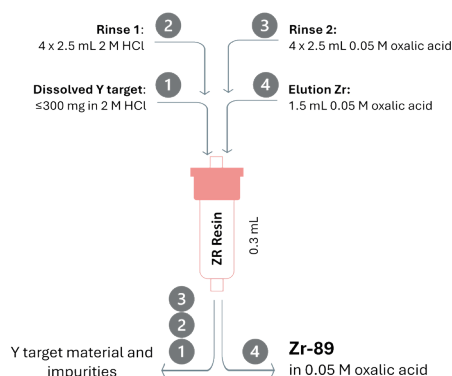
As indicated by the D_w values, the ZR Resin will retain Zr over a range of HCl concentrations. The rinsing conditions were kept close to the conditions suggested by Holland et al.: after loading the resin it is first rinsed with 4 x 2.5mL 2M HCl, followed by an additional rinse with 4 x 2.5mL water. Zr is finally eluted using 0.05M oxalic acid or higher.

Under the given conditions a very clean separation of Zr from Y, Ti and Fe was obtained. Y and Fe are removed during the loading and rinsing of the ZR Resin, while Ti remains retained on the resin. Zr can be recovered quantitatively in ~2 bed volumes (BV) of 0.05M oxalic acid. High chemical yields could be obtained even in presence of up to 300mg stable Y (using 100mg ZR Resin).



Elution study ZR Resin, 100mg, various elements, fractions analysed by ICP-MS

The ZR Resin is currently also being tested for use in radioanalytical applications such as the quantification of Zr-93, as well as, in combination with the TK400 Resin, the separation of Fe/Nb/Mo, e.g. in decommissioning samples.



Suggested method for the separation of Zr from Y targets (≤ 300mg) using the ZR Resin.

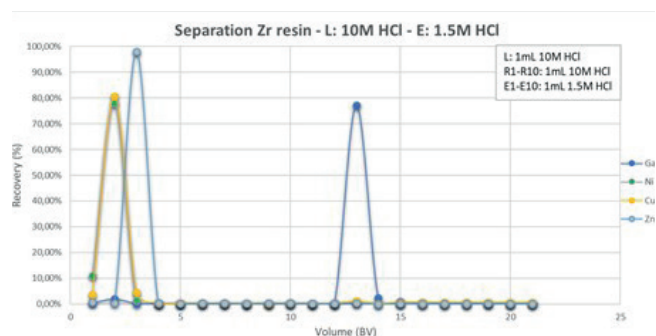
Another increasingly important use of the ZR Resin is the purification of Ga-68 from irradiated Zn targets. The production of Ga-68 via the irradiation of Zn-68 (as liquid or solid targets) on a cyclotron is indeed increasingly finding use as an alternative to Ge-68 generator produced Ga-68, as it allows frequent production of high activities.

As shown in the D_w value graphs the ZR Resin retains Ga very strongly in dilute HCl and HNO_3 , as well as in HCl of elevated concentration ($\geq 5\text{M}$ HCl). Zn on the other hand is not retained under any of these conditions. This selectivity allows its use for the separation of Ga-68 (and Ga-67) from irradiated Zn targets - liquid targets (typically dilute HNO_3) as well as solid targets (typically dissolved in HCl of high concentration).

HCl between 1M and 2M on the other hand is very suitable for Ga elution as its retention is particularly low under these conditions.

The following graph shows a typical example of such a separation. While Zn, and other typical impurities such as Cu and Ni, are not retained on the ZR Resin (in this example from high HCl) Ga is very well retained. A clean Ga fraction is then obtained by elution of Ga e.g. with 1.5M HCl.

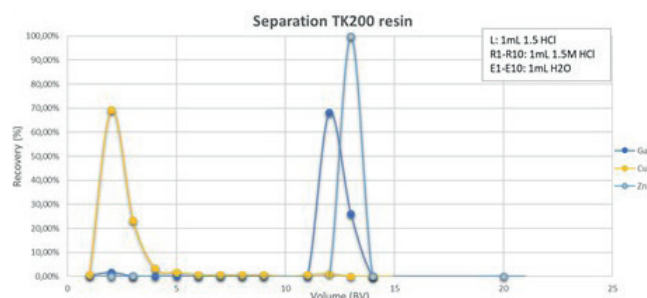
This Ga-68 containing 1.5M HCl solution is too acidic for direct use in labelling or injection. Instead of converting the solution to more suitable conditions e.g. via evaporation and redissolution it is possible to use another resin for this step, the TK200 Resin. More information on the TK200 Resin selectivity may be found in its product sheet, in this context the most important fact is that it retains Ga well in the range of 1 – 2M HCl, while it allows for Ga elution in dilute HCl or water, making it very suitable for this required conversion.



Elution study, Ga separation on ZR Resin, various elements, fractions analysed by ICP-MS

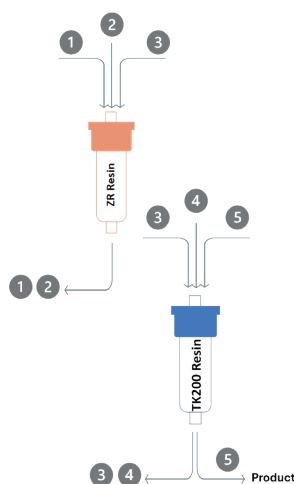
It should be noted though that while other impurities like Cu are very well removed during load and rinse on the TK200, remaining traces of Zn are only partially removed, accordingly a clean separation on the ZR Resin upfront is very important, as can also be seen in the following elution study.

Several publications describe the use of the combination of ZR and TK200 Resins for the separation of Ga-68 from liquid Zn targets. The publication by Rodnick et al. is particularly interesting as it describes the use of a modified rinse on the TK200 Resin cartridge (2M NaCl/0.13M HCl) instead of the usual rinse with HCl.



Elution study, Ga conversion on TK200 Resin, various elements, fractions analysed by ICP-MS

This allows for recovering the final Ga fraction at a better-defined HCl concentration during elution. The following scheme shows the separation method they developed.



		Reagent and volume
1	ZR Load	< 0.1 M HNO ₃
2	ZR Wash	15 mL 0.1 M HNO ₃
3	ZR Elution / Trapping on TK200	5-6 mL ~1.75 M HCl
4	TK Wash	*Previously reported: no wash Optimized: 3.5 mL 2 M NaCl in 0.13 M HCl
5	TK Elution	*Previously reported: H ₂ O Optimized: 1-2 mL H ₂ O followed by dilute HCl to formulate

* Process as reported previously (Nair et al. 2017)

Scheme of Ga-68 separation from liquid Zn targets using ZR Resin and TK200 Resin, taken from Rodnick et al.

Compared to liquid targets the irradiation of solid Zn targets allows for obtaining higher Ga-68 activities per production run.

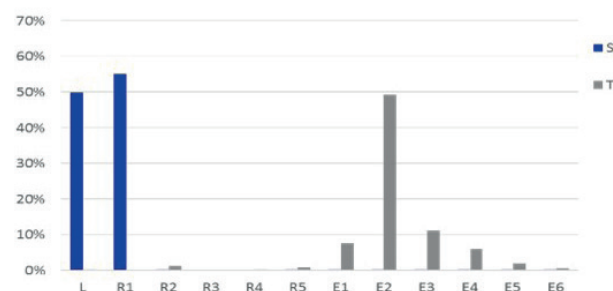
Thisgaard et al. describe the production of 194GBq Ga-68 (at end of purification) as [⁶⁸Ga]GaCl₃ of high purity, and it's subsequent successful use for the labelling of PSMA-11 and DOATATE. The authors used a three-resin method for the separation.

An additional LN Resin cartridge is used between the ZR Resin and the TK200 Resin to further remove potentially present impurities, particularly Fe. Svedjehed et al. successfully tested ZR/A8/TK200 Resins and TK400/A8/TK200 Resins combinations for solid Zn targets.

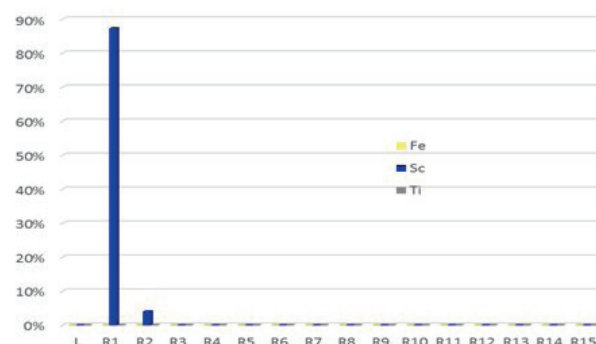
The ZR Resin further shows very interesting selectivity for Ti, especially with respect to Sc.

The graph below shows an example of a Ti/Sc separation performed on the ZR Resin, while Sc is not retained from 10M HCl Ti is fixed very well. 0.1M citric acid may then be used to elute Ti from the resin. However, the elution requires up to 10 bed volumes (BV) of the eluent or more. Beside citric acid, hydrogen peroxide or oxalic acid of elevated concentration may also be employed. As Ti is retained over a very wide range of HCl concentrations, including dilute HCl, its potential for use as support for a Ti/Sc generator was also evaluated initially. In order to do so a 100mg ZR Resin column (0.3mL) was loaded with a small volume of a solution containing Ti and Sc. The column was then rinsed five times with 1mL 0.01M HCl, followed by 10 rinses with 5mL 0.01M HCl. Sc is easily removed in a small volume of dilute hydrochloric acid whereas Ti remains retained throughout the experiment, the general selectivity of a generator is thus given.

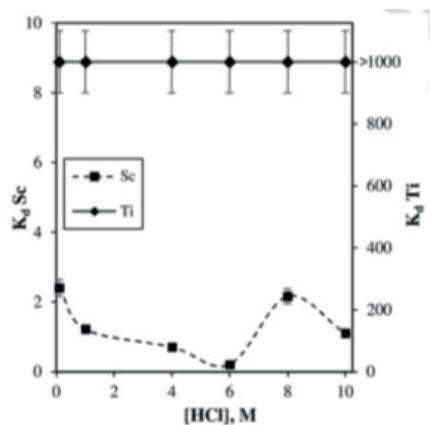
Radchenko et al. examined the system in greater detail and using real, irradiated samples. They confirmed the ZR Resins selectivity for Ti over Sc, shown by the k_D values the author obtained.



Ti/Sc separation on ZR Resin (0.3mL), fractions analysed by ICP-MS

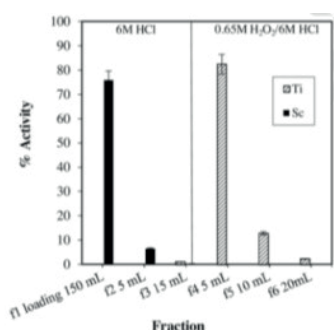


Elution study on 100mg ZR Resin, Ti and Sc, load from 0.01M HCl, repeated elutions, fractions analysed by ICP-MS



K_d values for Sc and Ti in HCl on ZR Resin, taken from Radchenko et al.

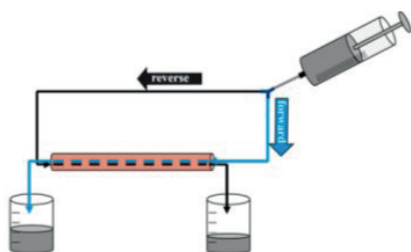
Based on these values they developed a separation method for the purification of Ti, to be more precise Ti-44, from irradiated Sc targets of elevated size (4g). The graph below shows the elution profile they obtained.



Ti/Sc elution profile on ZR Resin, irradiated 4g Sc target, taken from Radchenko et al.

As may be seen a clean Ti separation from Sc was obtained. The Ti-44 was obtained as an HCl/H₂O₂ solution.

The authors used the purified Ti-44 solution for the preparation of two types of Ti-44/Sc-44 generators, one direct flow generator and, the preferred option according to the authors, one 'forward/reverse flow' generator.



Schematic of a forward/reverse flow radionuclide generator, taken from Radchenko et al.

Especially the 'forward/reverse flow' generator showed very promising results with stable very low Ti breakthrough and high Sc elution yields. The obtained Sc-44 was successfully used to perform DOTA labelling with high yields, further indicating its high purity.

Malinconico et al. also used the ZR Resin to produce Ti-45 from irradiated Sc-45 targets.

Besides for the purification of Ga-68 from Zn targets the ZR Resin may actually also be used for the separation of Ge-68 from irradiated GaNi or GaCo targets. As discussed before, while Ga is very well retained at low mineral acid concentrations (typically $\leq 0.1M$) and at high HCl concentrations, it is not retained at medium high HCl and HNO₃ concentrations, and medium to high H₂SO₄ concentrations. Ge on the other hand is very well retained at elevated mineral acid concentrations. The D_w values show that especially between 1M – 3M HCl and HNO₃ the Ge retention is significantly higher than Ga retention.

It could be shown that the same is true for e.g. 5M H₂SO₄. The ZR Resin further shows no selectivity for Ni or Co under these conditions.

While the selectivity for Ge over Ga is very high in HCl its use for Ge separations is often avoided due to the high volatility of GeCl₄. In H₂SO₄ on the other hand Ge is not volatile, it further allows an efficient dissolution of typically employed target materials. Accordingly, a method for the separation of Ge-68 from multi gram irradiated GaNi or GaCo targets is currently being optimized.

The method is based on two subsequent purification steps on ZR Resin. First the dissolved target is adjusted to 5M H₂SO₄ and then loaded onto a 2mL ZR Resin cartridge. After rinsing with 5MH₂SO₄ and purge with air for acid removal Ge is eluted with dilute citric acid. The Ge fraction is again adjusted to 5M H₂SO₄ and further purified on a 1mL ZR Resin cartridge. Ge is once more recovered in dilute citric acid. In order to obtain the final product in dilute HCl (typically 0.05M HCl) the Ge is converted from citric acid to dilute HCl by adjusting it to 9M HCl, loading onto a Guard Resin cartridge, followed by elution with water or dilute acid. Further optimisation of the method is currently on-going.

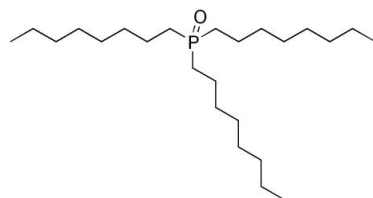
Main applications



- Separation of zirconium
- Separation of gallium
- Separation of germanium
- Separation of titanium

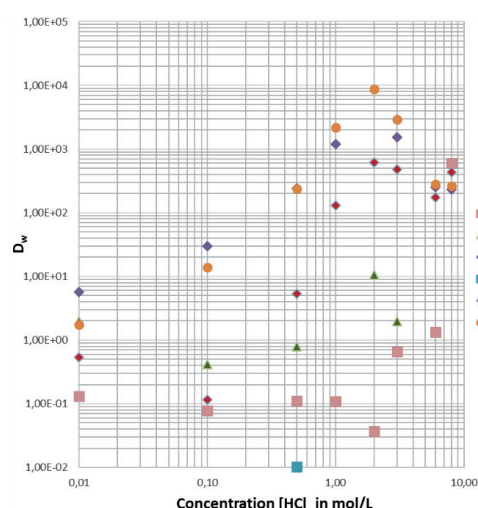
TK200 Resin

The TK200 Resin is based on TriOctylPhosphine Oxide (TOPO) an extractant widely used in the extraction of metal ions.



Trioctylphosphine oxide (TOPO)

Some examples of D_w values determined in HNO_3 and HCl using ICP-MS are shown below.



D_w values of selected elements on TK200 Resin in HCl

Cd, Zn and Ga are very well retained at HCl concentrations $>1\text{M}$. This is especially interesting with respect to Ga separation chemistry as Ga is not retained at 1 – 2M HCl on most resins.

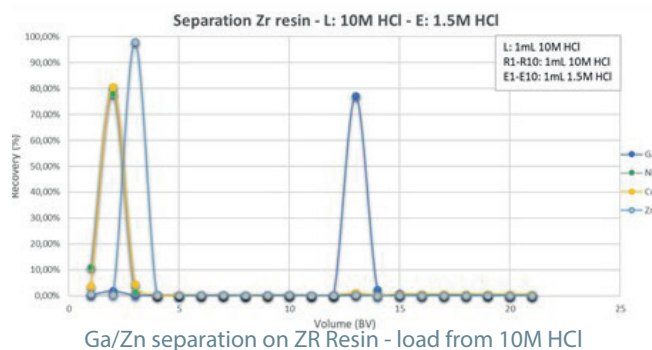
In HNO_3 none of these elements are retained including e.g. Ga and Zn.

D_w values for a wide range of additional elements may be found in the corresponding product sheet.

A typical example of the use of the TK200 Resin is the separation of Gallium isotopes (especially Ga-68) from irradiated Zn targets for medical use in combination e.g. with the ZR Resin.

ZR Resin is very well suited for the separation of Ga from Zn matrices, under low acid conditions (e.g. 0.1M HNO_3 often employed for liquid targets) as well as at high acid concentrations (e.g. $>5.5\text{M}$ HCl conditions often used for the dissolution of solid Zn targets).

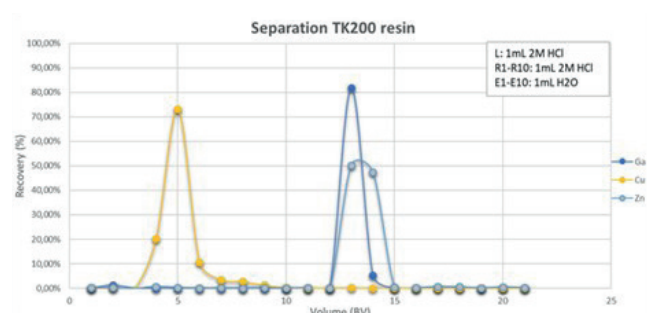
The following elution study shows the separation of Ga from Zn and potential impurities on ZR Resin.



Ga/Zn separation on ZR Resin - load from 10M HCl

Ga is eluted from the ZR Resin in a small volume (1 – 2 column volumes) of 1.5M HCl, conditions too acidic for direct use in labelling reactions.

The TK200 Resin on the other hand allows for Ga extraction at 1.5M HCl, followed by Ga elution using aqueous solutions.



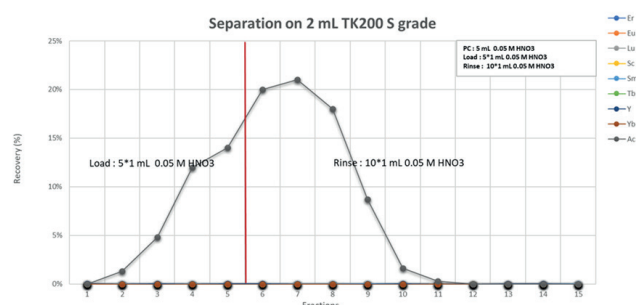
Ga elution from TK200 Resin with water following load from 1.5M HCl

It should be noted though that only very little additional Ga/Zn separation is taking place on the TK200 Resin.

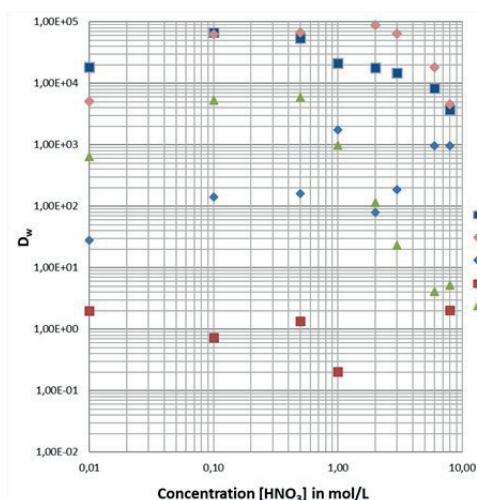
Another typical application of the TK200 Resin is the determination of actinides such as U, Th and Pu in water samples.

As shown in the following figure, U and Th are very well retained over the whole HNO_3 concentration range, including 0.01M.

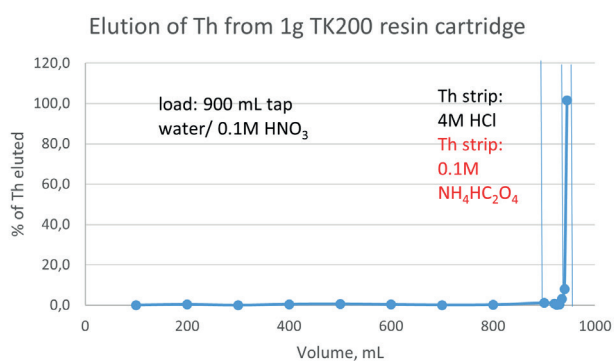
Another interesting aspect of the TK200 Resin is that it shows no retention of Ac from 0.05M HNO_3 while other alpha emitters such as U and Th are very well retained, accordingly it might find use in Ac purification and QC.



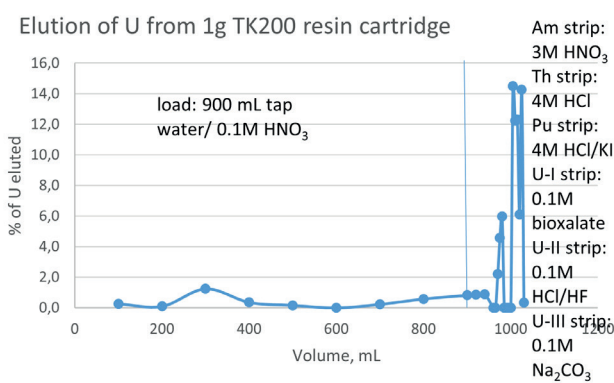
Elution study, TK200 Resin, 0.05M HNO_3 , Ac and selected elements



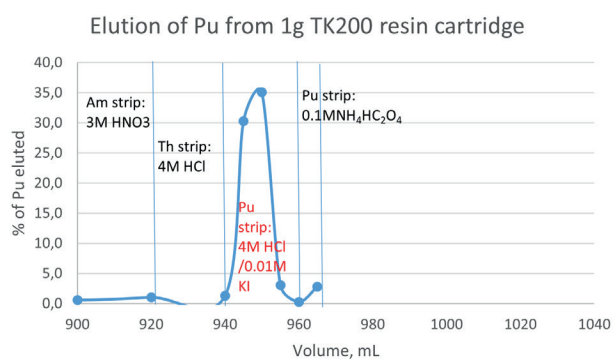
D_w values of selected elements on TK200 Resin in HNO_3



Elution study Th retention and elution on TK200 Resin (data courtesy of Nora Vajda)



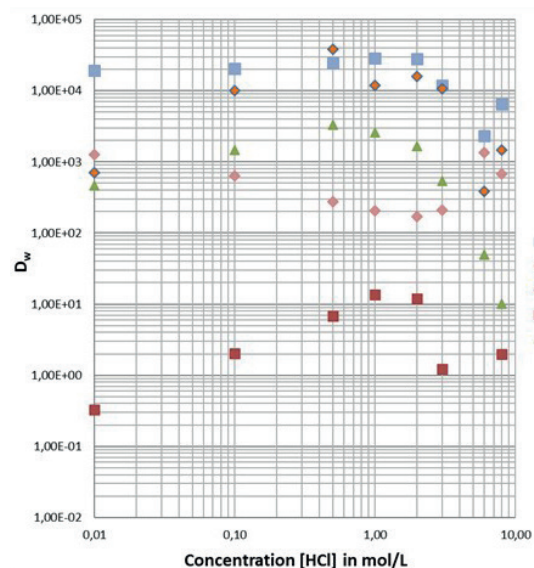
Elution study U retention and elution on TK200 Resin (data courtesy of Nora Vajda)



Elution study Pu retention and elution on TK200 Resin (data courtesy of Nora Vajda)

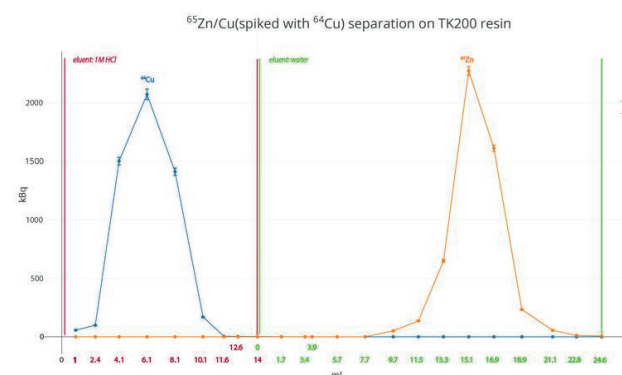
This allows for using the TK200 Resin for the preconcentration of actinides from acidified water samples, and their subsequent separation on the same column.

U and Pu are also very well retained from HCl.



D_w values of selected elements on TK200 Resin in HCl

A number of other methods are currently being tested, including the separation of Pd from Rh targets, the separation of Pt from Ir targets and Zn from Cu targets.



Zn-65 separation. Data provided by Fedor Zhuravlev, DTU

Main applications

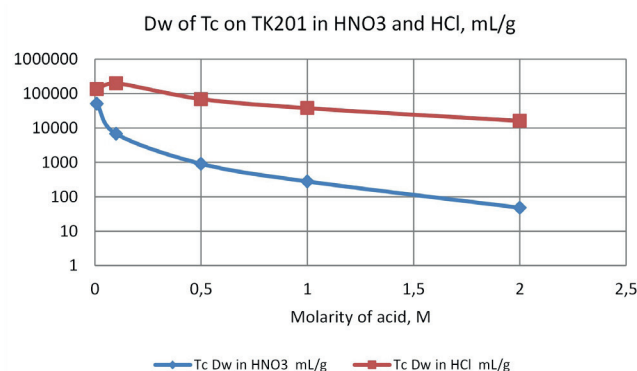
- Separation of gallium
- Ac purification
- Purification of lanthanides

TK201 Resin

The TK201 Resin is based on a tertiary amine, it further contains a small amount of a long-chained alcohol (radical scavenger) to increase its radiolysis stability. The TK201 Resin acts as a weaker ion pair binding agent compared to the TEVA Resin, accordingly it is generally possible to elute it under softer conditions.

Its main applications are the separation of anionic species such as Tc(VII) or Re(VII) and the separation of Cu isotopes from solid Ni targets.

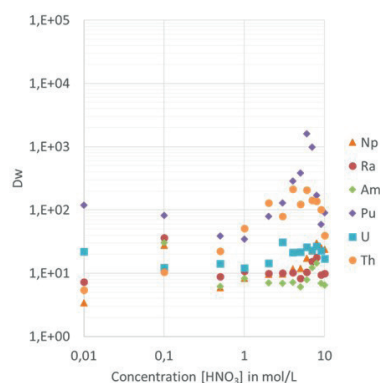
The following graph shows the D_w values for Tc in HNO_3 and HCl.



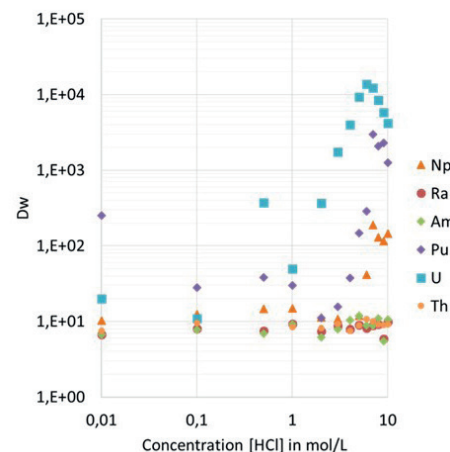
D_w values of Tc on TK201 Resin in HCl and HNO_3 , obtained by LSC, data provided by N. Vajda (RadAnal)

Tc(VII) is very well retained at low acid concentrations. Its retention is generally significantly higher in HCl than in HNO_3 , even at elevated HCl concentration such as 2M it remains very strongly retained. In HNO_3 on the other hand its retention is rather low at concentrations above 2M.

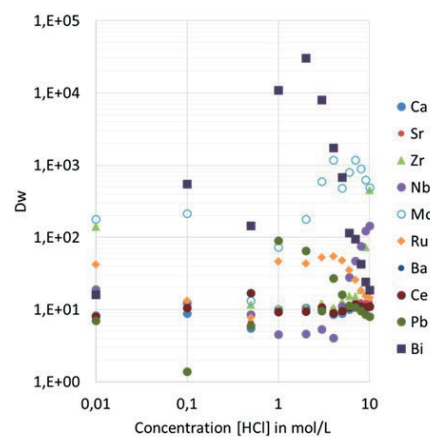
The following graphs show the selectivity of the TK201 Resin for a wide range of elements in HCl and HNO_3 . All D_w shown in these graphs were obtained through ICP-MS measurements.



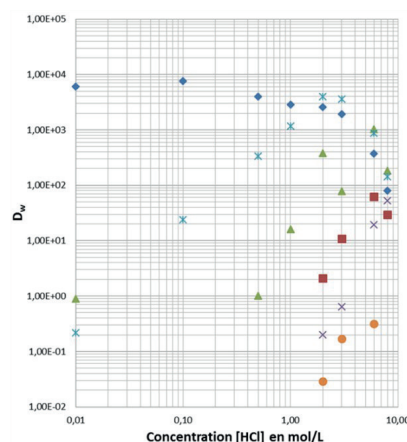
As expected, the TK201 Resin shows strong Re(VII) retention in HCl even at high concentrations. Further Zn, Ga and Cu are retained, especially the latter allows for its use in radiopharmaceutical applications.



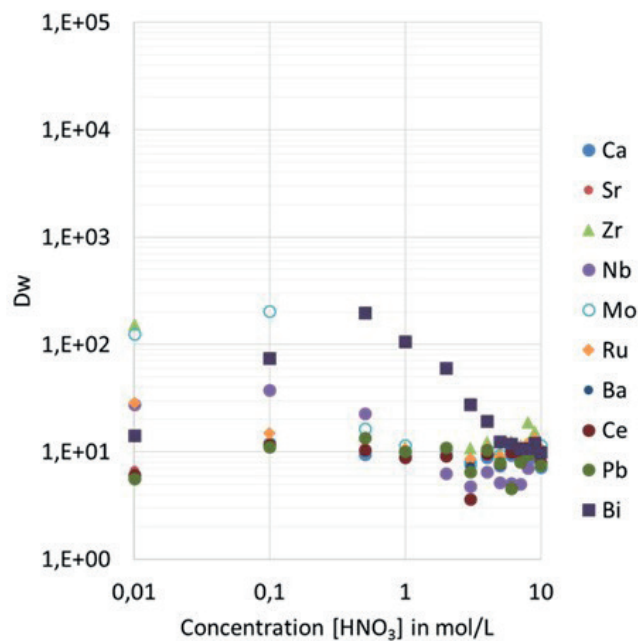
The TK201 Resin also shows strong retention of U and Pu at elevated HCl concentrations, both might subsequently be eluted in dilute acid.



The TK201 Resin further strongly retains Bi and Mo at elevated HCl concentrations, while other elements tested show no or only very low retention (Ru, Nb).



The TK201 Resin generally shows rather limited selectivity in HNO_3 , besides for Re and Tc at low acid concentration. At elevated HNO_3 concentrations Pu is well retained and Th fairly well, other actinides are not retained under these conditions.

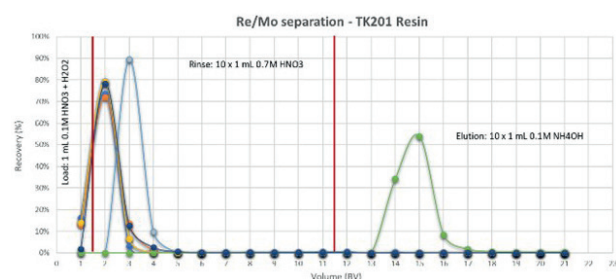


D_w values of selected elements on TK201 Resin in HNO_3 , data provided by Russel et al. (NPL)

Out of the other elements tested only Bi (at about 0.5M HNO_3) and Mo (at low HNO_3 concentrations) are retained significantly. It is important to note that Mo is not retained at HNO_3 concentrations above 0.5M while Tc and Re are well retained (shown on the 1st figure), allowing for their clean separation.

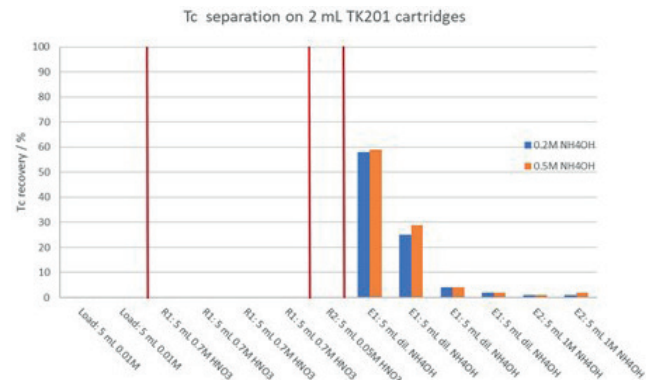
It could further be shown by Vajda et al. that D_w values for Tc(VII) are very low in dilute NH_4OH : in 0.1M NH_4OH Tc(VII) shows a D_w of only ~2, accordingly it is easily eluted by $\geq 0.1\text{M}$ NH_4OH .

Additional elution studies indicated that an efficient Mo separation from Re is possible using 0.7M HNO_3 for Mo removal and dilute NH_4OH for Re elution.



Elution study, Re separation from various elements (incl. Mo and W).

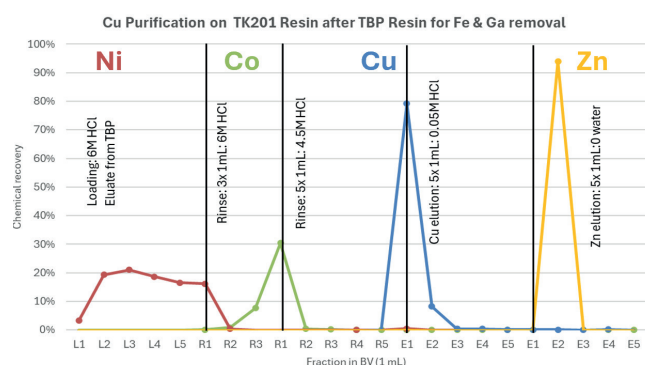
Vajda et al. could confirm that Tc is, like Re, not eluted in 0.7M HNO_3 , validating that Re is a good surrogate for Tc and thus also allowing an efficient Mo/Tc separation. Most suitable conditions for Tc elution were found to be NH_4OH greater or equal to 0.2M.



Elution study, Tc separation on 2mL TK201 Resin cartridges, data provided by N. Vajda (RadAnal)

The other main application of the TK201 Resin is the separation of Cu isotopes (e.g. Cu-64) from solid Ni targets. Other than the CU Resin the TK201 Resin allows for Cu retention from high HCl (e.g. 6M), while letting Ni pass for subsequent recycling. Other potential impurities (e.g. Co) may be removed through rinses with 4 – 5M HCl. Cu may then be eluted in dilute HCl leaving Zn on the column.

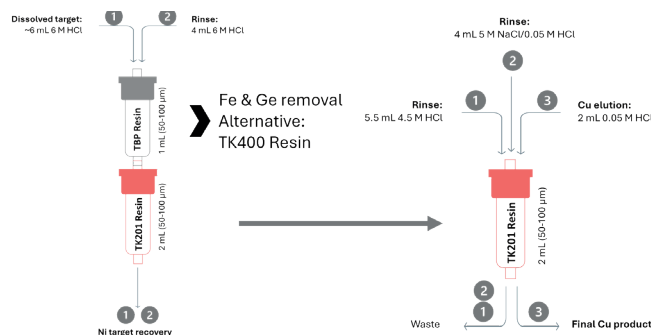
A typical separation is shown in the graph below.



Elution study, Cu separation on 1mL TK201 cartridges

In order to remove Fe and Ga impurities potentially present the dissolved Ni target (6M HCl) may first be loaded through a small TBP (or TK400) Resin cartridge which will retain both elements while letting Ni, Cu and Zn pass onto TK201 Resin for further purification. Cu may then be eluted from TK201 Resin e.g. in 0.05M HCl.

This could be demonstrated i.e. by Svedjehed et al. The rinse with 5M NaCl/0.05M HCl is particularly noteworthy as it allows obtaining the final product in dilute HCl solution of defined concentration.



Cu separation using TBP and TK201 Resins according to Svedjehed et al.

Main applications

- Separation of Cu isotopes from solid Ni targets
- Separation of technetium
- Separation of rhenium

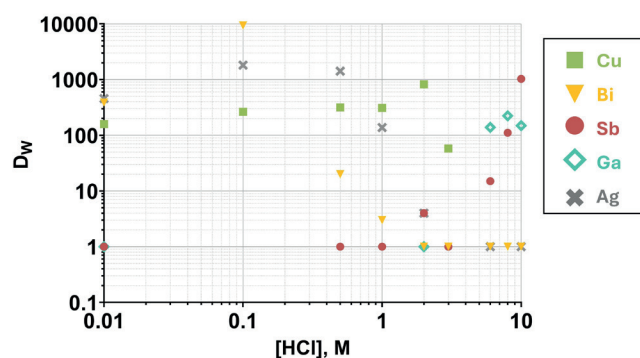
TK201 Resin may also be used to convert the Cu fraction eluted from the CU Resin (e.g. for the separation of Cu isotopes from Zn targets) from a highly acidic solution (e.g. 6 - 8M HCl) to conditions more suitable for labeling (e.g. dilute HCl). TK201 Resin will retain Cu e.g. from 6M HCl and can then be eluted with dilute HCl as shown e.g. by Kawabata et al. This step will also ensure further Zn removal.

TK250 Resin

The TK250 Resin is an extraction chromatographic resin based on an extractant that shows high selectivity for Cu over transition metals even at rather elevated acid concentration. It should be noted that the TK250 Resin shows a rather low Cu capacity compared to other resins (in the order of 200µg Cu per g of TK250 Resin).

Its main application is the separation of Cu isotopes from irradiated Ni or Zn targets.

The two graphs below show the selectivity of the TK250 Resin in HCl and HNO₃. All D_w values shown in these graphs were obtained through ICP-MS measurements.

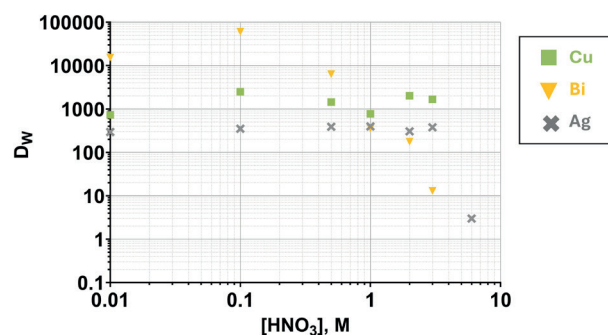


D_w values, TK250 Resin, selected elements, HCl

From HCl only Cu, Bi, Sb, Ga and Ag are retained. While elements such as Cu, Ag and Bi are preferably retained at lower HCl concentrations such as 3M HCl or less for Cu, and 1M HCl and less for Ag, Ga and Sb show an opposite selectivity, they are only retained under very high HCl concentration, conditions where Cu, Ag and Bi are typically eluted.

Other elements tested, including Zn, Ni, Fe and Co are not retained in HCl. The TK250 Resin thus allows for the separation of Cu from these elements

even at rather elevated HCl concentrations (1 – 3M). This is especially relevant in case of Zn, as this allows the separation of Cu from Zn targets without the pH adjustment required by the CU Resin, otherwise the method of choice especially for solid Zn targets.

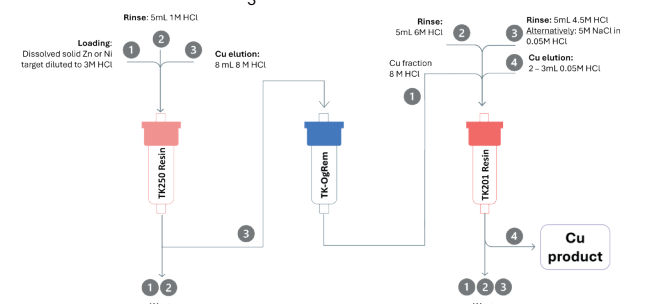


D_w values, TK250 Resin, selected elements, HNO₃

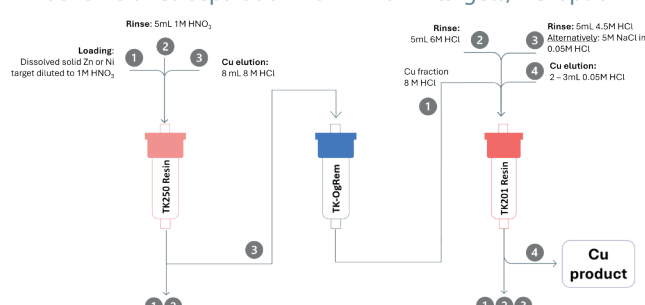
Cu and Ag show quite elevated D_w values up to 3M HNO₃. That said, on-column tests showed that it is preferable to limit the HNO₃ concentration to 1M HNO₃. The fact that the TK250 Resin allows for Cu separation from Ni and Zn targets in 1M HNO₃ sets it apart from the TK201 Resin (separation of Cu from solid Ni targets in HCl) and the CU Resin (separation of Cu from Ni or Zn targets at pH >2). This might be of interest as some targets are easier to dissolve in HNO₃. The elution of Cu requires in both cases acid of elevated concentration (preferably 8M HCl). These conditions are not compatible with a direct labelling of the purified Cu isotope; it is thus necessary to convert this solution to dilute HCl before use. While evaporation and redissolution is also possible, these steps are difficult to automate. The use of a TK201 cartridge for this conversion, similar to the methods described by Nagai et al. and Svedjehed et al. presents an easier to automate option.

It should be noted that it is further necessary to remove organic material bleeding from the TK250 Resin during elution before loading the Cu fraction onto TK201. This is typically done using a 2mL TK-OgRem cartridge. Not doing so will result in irreversible retention of Cu on the TK201 cartridge, and thus low chemical yields.

The scheme below shows a typical method for the separation of Cu from Ni or Zn targets in HCl, and the following scheme a method for the same separation but with a load and rinse in 1M HNO₃.

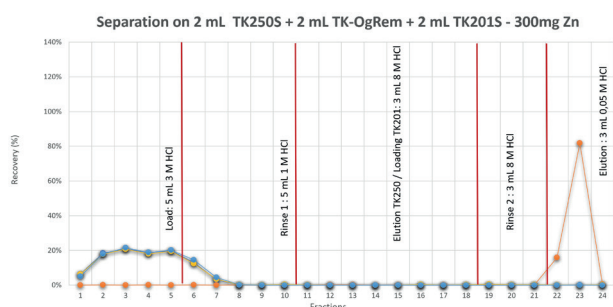


Scheme of Cu separation from Zn or Ni targets, HCl option

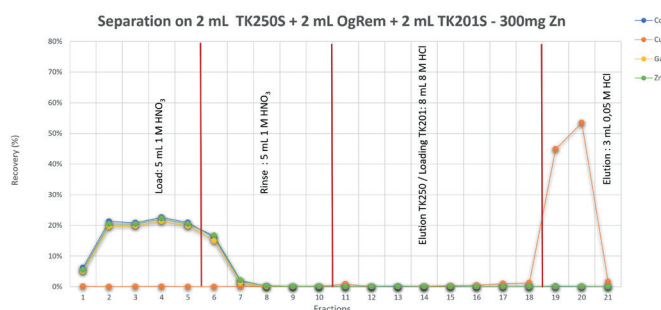


Scheme of Cu separation from Zn or Ni targets, load and rinse in 1M HNO₃

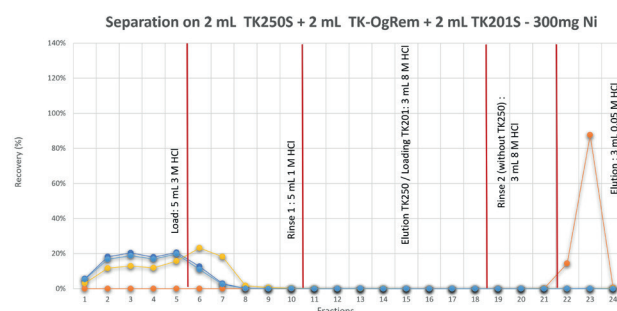
The four graphs below show typically obtained chromatograms using these two method with the first two graphs showing the separation in HCl, and the following two in HNO₃.



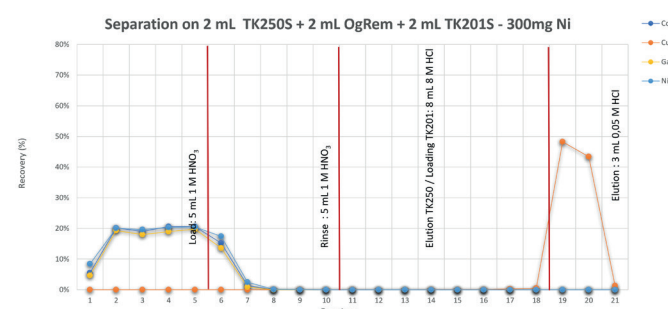
Elution study, simulated 300mg Zn target, load from 3M HCl, rinse with 1M HCl, elution in 8M HCl and conversion on TK201



Elution study, simulated 300mg Zn target, load from 1M HNO₃, rinse with 1M HNO₃, elution in 8M HCl and conversion on TK201



Elution study, simulated 300mg Ni target, load from 3M HCl, rinse with 1M HCl, elution in 8M HCl and conversion on TK201

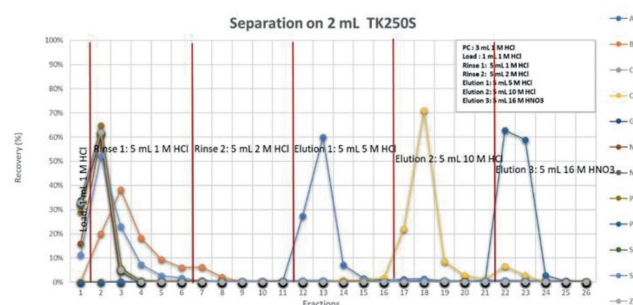


Elution study, simulated 300mg Ni target, load from 1M HNO₃, rinse with 1M HNO₃, elution in 8M HCl and conversion on TK201

It should be noted that especially in HNO₃ a colour change of the TK250 Resin may occur.

Both options allow for good Cu yields and high removal of target materials and other impurities potentially present.

TK250 Resin further showed selectivity for Pd, which opens up the possibility of a separation of Ag, Cu and Pd, as shown below.



Elution study, multielement solution incl. Cu, Ag and Pd

Main applications

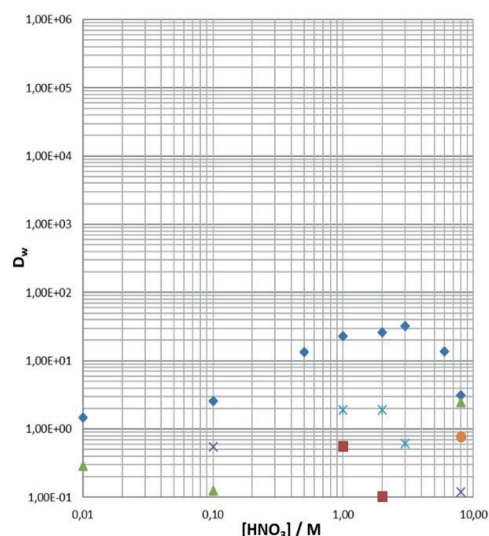
- Cu separation from Ni and Zn targets

TK221 Resin

The TK221 Resin is based on a mixture of a diglycolamide and a phosphine oxide. It further contains a small amount of a long-chained alcohol and the organic phase is impregnated onto an inert support containing aromatic groups for increased stability against radiolysis.

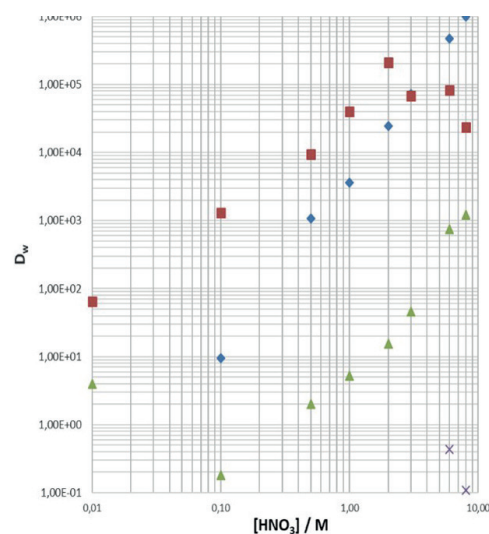
The following graphs show the selectivity of the TK221 Resin for a wide range of elements in HNO_3 and HCl .

Out of the tested elements only Ca is weakly retained on the TK221 Resin in HNO_3 . Other alkaline, earth alkaline elements and Al are not retained.



D_w values of selected elements on TK221 Resin in HNO_3

Y and Sc are very strongly retained from HNO_3 of elevated concentration. Fe(III) is also well retained at HNO_3 concentration $\geq 3\text{M}$ HNO_3 .

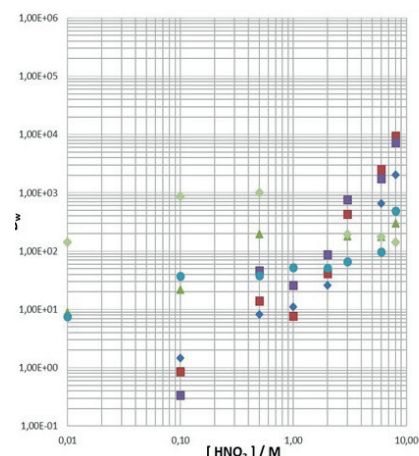


D_w values of selected elements on TK221 Resin in HNO_3

A wide range of transition metals such as Zn, Ga, Co, Ni and Cu are not retained from nitric acid.

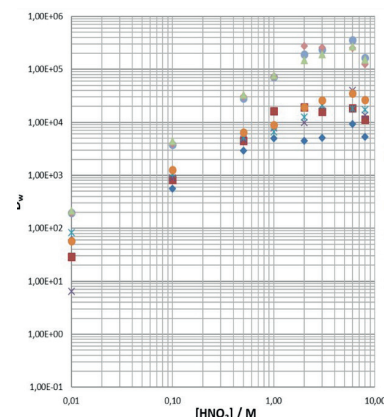
The TK221 Resin generally retains tetravalent elements such as Zr and Hf at elevated HNO_3 concentrations.

The TK221 Resin shows very high retention of lanthanides at HNO_3 concentrations $\geq 0.1\text{M}$ HNO_3 , heavy lanthanides are even well retained in more dilute HNO_3 ($\geq 0.01\text{M}$). The retention of the lanthanides is significantly stronger than on TRU Resin.

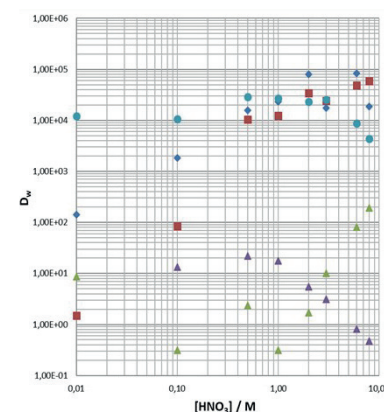


D_w values of selected elements on TK221 Resin in HNO_3

U and especially Bi are well retained over the whole HNO_3 concentration range, while Th is well retained at $\text{HNO}_3 > 0.1\text{M}$. U retention is significantly higher than on other diglycolamide based resins such as DGA Resin. Pb and Sn are only weakly retained.



D_w values of selected elements on TK221 Resin in HNO_3

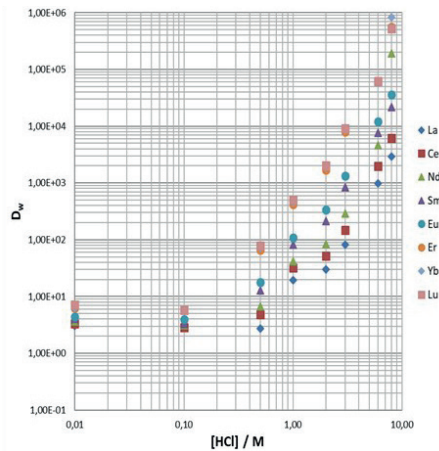


D_w values of selected elements on TK221 Resin in HNO_3

In HCl medium, none of the tested alkaline and earth-alkaline elements were retained on the TK221 Resin the same is true for Al.

Other than many other transition metals, Zn and Ga are very well retained from $\geq 2\text{M}$ HCl. Both may be easily eluted in dilute HCl.

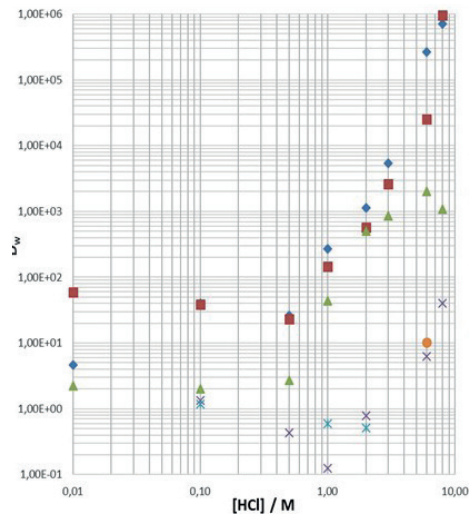
U, Sn and Bi are well retained over the whole HCl concentration range, while Th is only well retained at $\geq 3\text{M}$ HCl. Pb is generally only very weakly retained.



D_w values of selected elements on TK221 Resin in HCl

Lanthanides are generally very well retained at HCl concentrations $\geq 3\text{M}$ HCl, heavy lanthanides even at $\geq 1\text{M}$, and they may be eluted in dilute HCl.

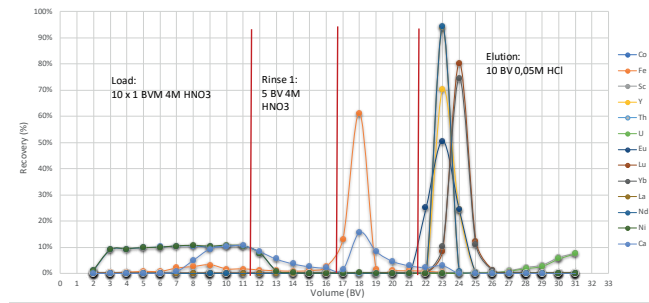
Like the lanthanides Sc, Y and Fe are also very well retained at high HCl concentrations, and eluted in dilute HCl.



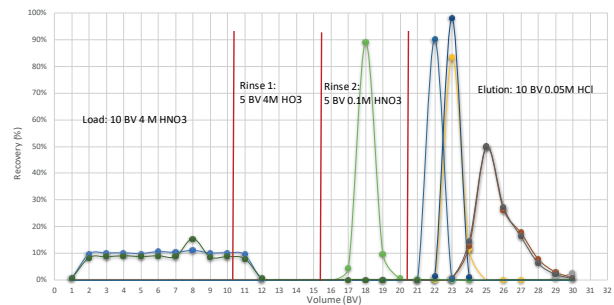
D_w values of selected elements on TK221 Resin in HCl

One of the main applications of TK221 Resin is the concentration, purification and conversion of heavy lanthanides such as Lu from highly acidic solutions into dilute HCl (typically $\leq 0.05\text{M}$ HCl) conditions.

It allows e.g. to elute Lu in a smaller volume than DGA,N Resin. Accordingly, it may e.g. find use in the production of Lu-177.



Elution study, various elements on TK221 Resin



Elution study, various elements on DGA, normal Resin

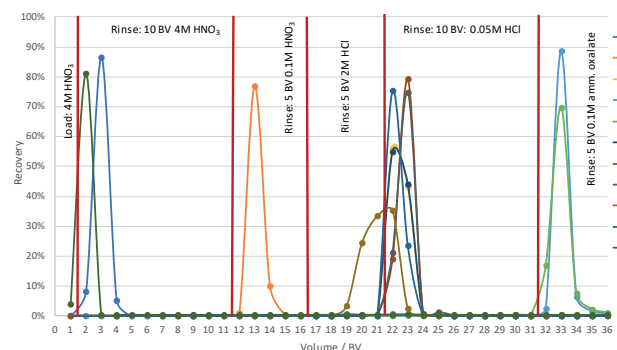
A number of separation methods based on the TK221 Resin are currently being developed particularly for c.a. and n.c.a. Lu-177 purification, as well as the use of TK221 as part of the separation of n.c.a. Lu-177 from up to 500mg Yb-176, and higher.

The final product obtained using the TK221 Resin in the latter separation is typically additionally passed through a 1mL A8 cartridge for trace nitrate removal.

Such a separation is also applicable to the purification of Ac-225.

The fact that the TK221 Resin is showing higher U retention compared to e.g. DGA,N Resin might further allow for its use in a two column separation method for sequential actinides separation.

The following figure shows an elution study of several elements including U on TK221.



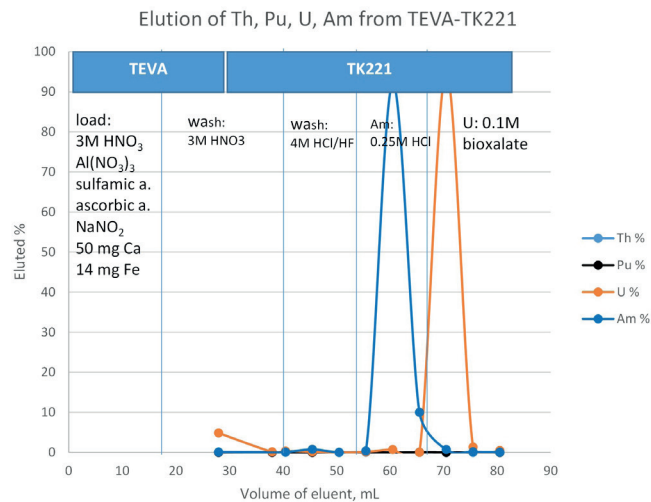
Elution study, various elements on TK221 Resin

U is very well retained under all employed HNO_3 and HCl concentrations and may finally be eluted in 0.1M oxalate. Am is expected to be eluted before U in dilute HCl .

With respect to the TK221 Resin selectivity a stacked TEVA/TK221 method for the separation of U, Th, Pu, Am/Cm and Np is very well possible.

In such a case Np(IV) , Pu(IV) and Th(IV) will be retained, and separated, on TEVA Resin while U and Am will pass through TEVA Resin onto TK221 Resin where both will be retained. It is then be possible to first elute Am with dilute HCl and finally U with dilute oxalic acid.

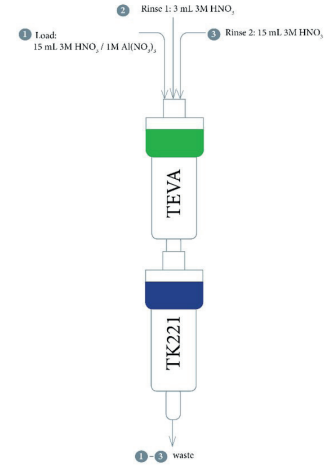
N. Vajda et al. developed such a method for the separation of Th, Pu, Am and U from water samples based on the TEVA/TK221 system. The development work was based on a typical Ca-Phosphate preconcentration step, and took into account the possible presence of Fe(III) originating from the oxidation state adjustment. Through a very thorough optimisation of the Am elution volumes a clean separation of Am and U on the TK221 Resin could be achieved as shown below.



Th, Pu, Am and U separation on 2mL TK221 Resin cartridge (data courtesy of N. Vajda et al.)

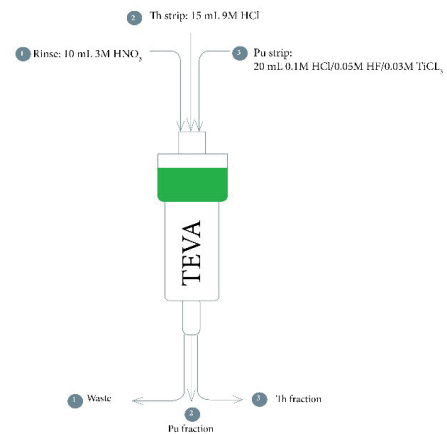
Overall, the authors reported high chemical yields (92 – 106%) under the given conditions, and very good decontamination of the obtained actinide fractions (cross-contamination <1% respectively).

The developed separation protocol is summarized in the following figures.

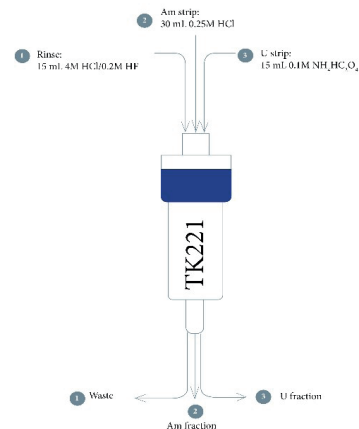


Th, Pu, Am and U separation on TEVA/TK221 Resins, stacked cartridges (according to N. Vajda et al.)

The dissolved Ca-Phosphate precipitate is first passed through stacked TEVA and TK221 Resin cartridges. The cartridges are then rinsed with 3M HNO_3 to assure matrix removal and quantitative transfer of U and Am onto the TK221 Resin cartridge. Both cartridges are then separated: Pu and Th are separated on the TEVA Resin cartridge, while U and Am are separated on the TK221 Resin cartridge.



Separation steps on split TEVA Resin cartridge (according to N. Vajda et al.)

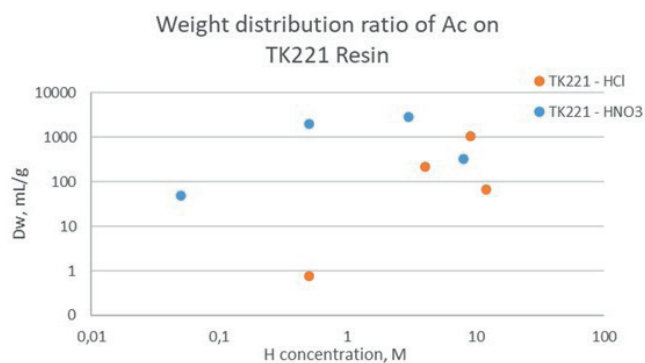


Separation steps on split TK221 Resin cartridge (according to N. Vajda et al.)

When applying the developed method to the alpha spectrometric determination of tap and a sea water samples spiked with Th-230, Pu-239, Am-241 and U-233 they could confirm the clean separation of the actinides, as well as the fact that high chemical yields may be obtained.

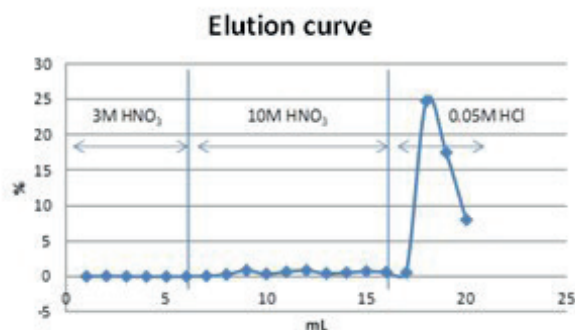
Even for a highly charged matrix such as a sea water sample, chemical yields were in the order of ~90% for U, Pu and Am and ~70% for Th, only about 10 – 20% lower compared to the tap water samples (90 – 108%), making this a very promising alternative to the classical TEVA/TRU methods with the additional benefit of a more robust Am retention.

N. Vajda et al. further examined the TK221 Resin with respect to Ac retention.



Weight distribution ratios (D_w) of Ac on TK221 Resin. Data courtesy of N. Vajda (RadAnal)

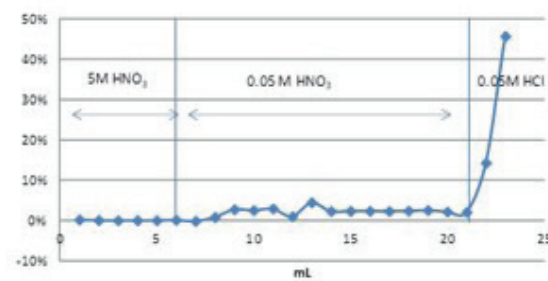
Overall, the D_w values for Ac are elevated in HNO_3 , even at very high (e.g. 8 - 10M) or low (e.g. 0.05M) concentrations. In HCl Ac retention is high at elevated HCl concentrations (9M HCl) while decreasing significantly at higher and lower acid concentrations. Especially at low HCl concentrations Ac D_w values are very low, indicating suitable elution conditions. The high retention of Ac over a wide HNO_3 concentration range could further be shown through elution experiments.



Ac elution study on 1mL TK221 Resin cartridge (100 – 200 μm), 10M HNO_3 rinse. Data courtesy of N. Vajda et al.

Other than the DGA Resins the TK221 Resin requires 12M HNO_3 for Ac elution. Under these conditions a clean Ac/La separation is achieved on TK221 Resin, which is generally not possible on DGA Resin.

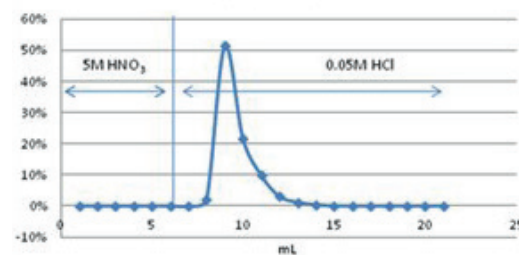
Elution curve



Ac elution study on 1mL TK221 Resin cartridge (100 – 200 μm), 0.05M HNO_3 . Data courtesy of N. Vajda et al.

TK221 Resin does not allow, contrary to the DGA Resins, Ac elution in dilute HNO_3 .

Elution curve



Ac elution study on 1mL TK221 Resin cartridge (100 – 200 μm), 0.05M HCl elution. Data courtesy of N. Vajda et al.

The fact that TK221 Resin retains Ac from dilute HNO_3 and allows its elution in dilute HCl might open the possibility of converting Ac solutions from dilute HNO_3 to dilute HCl using TK221 Resin, avoiding evaporation of the Ac solution..

It should be noted that TK221 Resin is now also available in 50 – 100 μm particle size now, which allows for narrower elution.

Overall, a separation method similar to Lu-177 purification seems possible: Ac retention from elevated HNO_3 , (or HCl) rinse with dilute HNO_3 to remove impurities and lower HNO_3 concentration on the resin, followed by elution in HCl. Ideally the final product should be loaded through a small anion exchange resin cartridge (e.g. 1mL A8 Resin) to remove last traces of nitrate.

Additional work on the Ac separation on TK221 Resin is currently on-going.

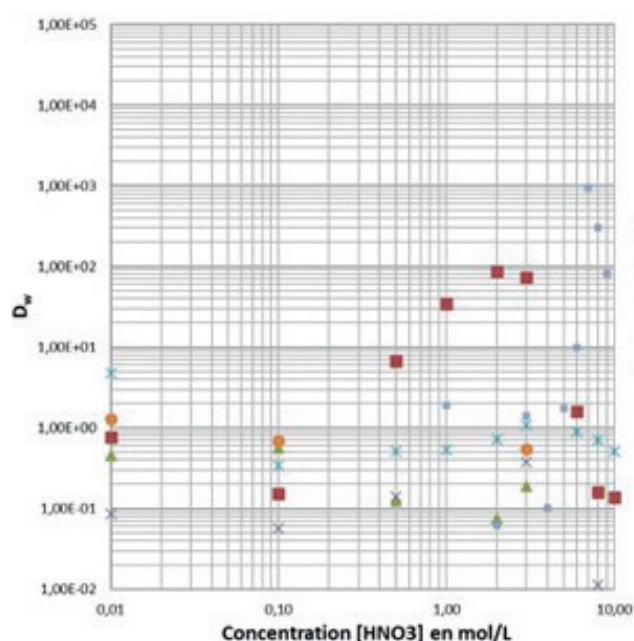
Main applications

- Separation and concentration of lanthanides (e.g. c.a. and n.c.a. Lu-177)
- Separation of actinides
- Separation of actinium

TK222 RESIN

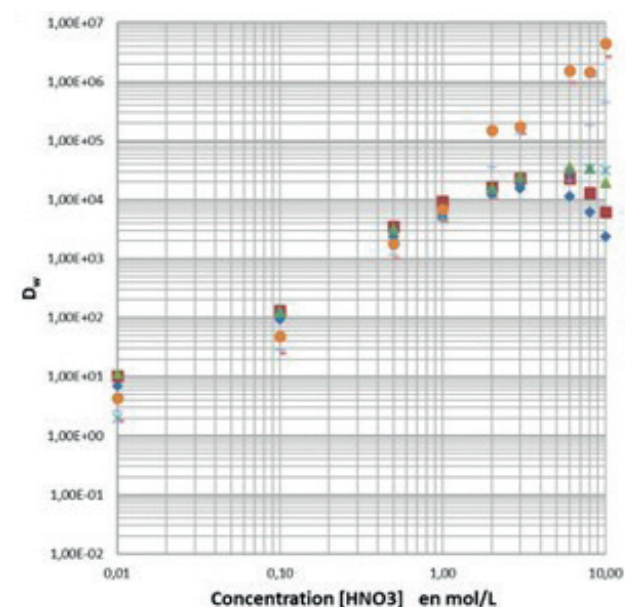
The TK222 Resin is based on a mixture of a branched diglycolamide and a phosphine oxide. It also contains a small amount of a long-chained alcohol. Further, the organic phase is impregnated onto an inert support containing aromatic groups for increased stability against radiolysis.

The following graphs show the selectivity of the TK222 Resin for a wide range of elements in HNO_3 and HCl . All D_w values shown in these graphs were obtained through ICP-MS measurements.



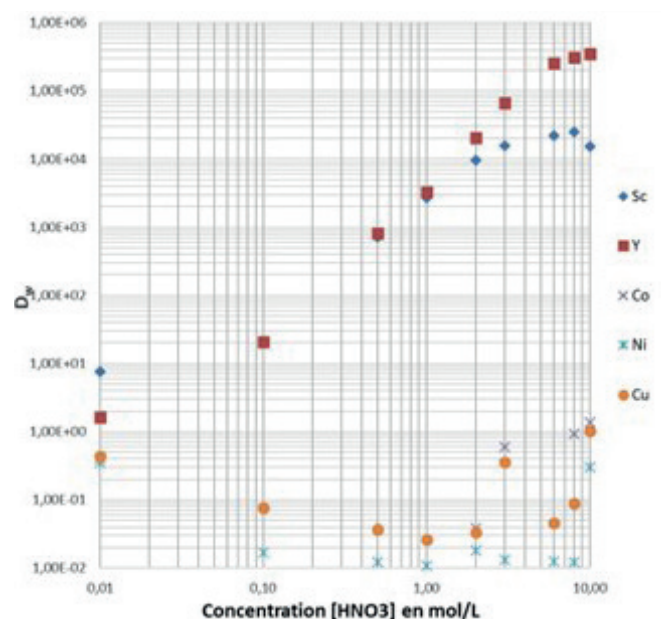
D_w values of selected elements on TK222 in HNO_3

Out of the shown elements only Sr at medium high HNO_3 concentration (2 – 3M) and Tl at elevated concentrations (~8M) are retained.



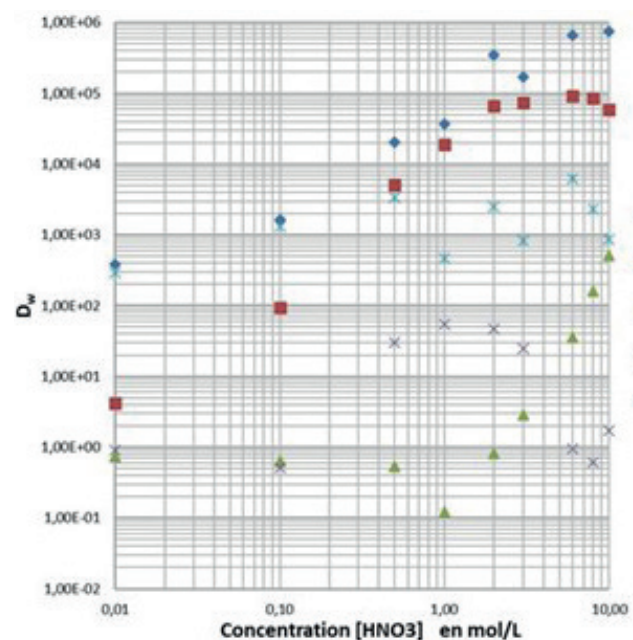
D_w values of selected elements on TK222 in HNO_3

Lanthanides are generally very well retained at elevated HNO_3 concentrations ($\geq 0.5\text{M}$), this is particularly true for heavy lanthanides. This point is particularly interesting with respect to the separation of lanthanides from Ac. D_w values are generally low at low HNO_3 concentrations.



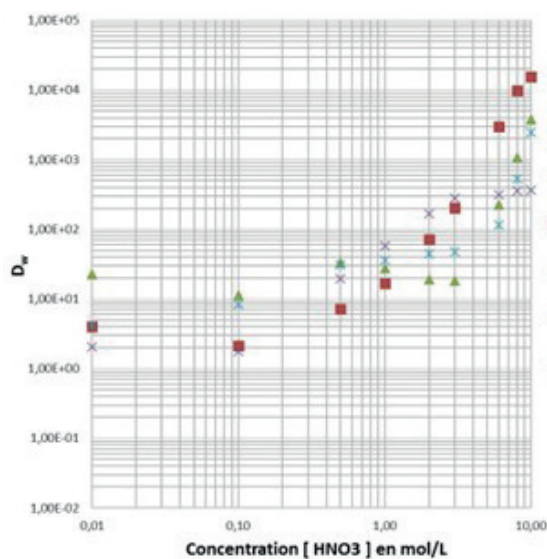
D_w values of selected elements on TK222 in HNO_3

Y and Sc are very well retained at elevated HNO_3 concentrations, while Co, Ni and Cu are not retained.



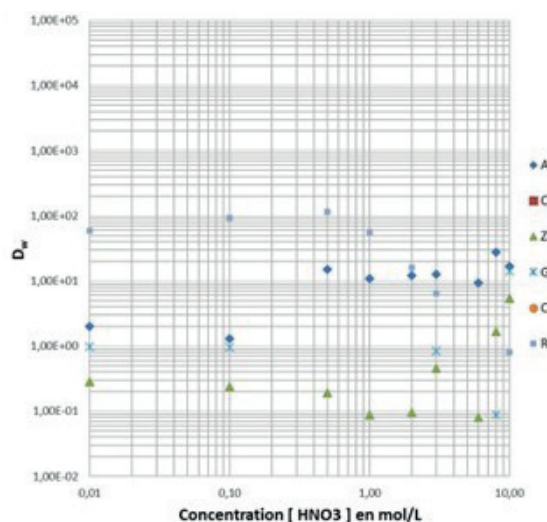
D_w values of selected elements on TK222 in HNO_3

U and Th are very well retained from elevated HNO_3 concentrations. Bi, too is well retained, to a lesser extent than U and Th though. Sn shows some retention at elevated HNO_3 . Pb is generally only rather weakly retained with a maximum between 0.5 and 3M HNO_3 .



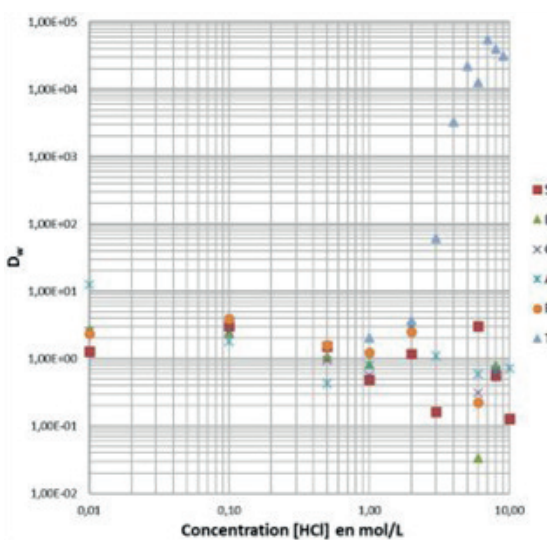
D_w values of selected elements on TK222 in HNO_3

Elements of higher valency such as Hf, Zr, Nb and Mo are well retained from HNO_3 of high concentration.



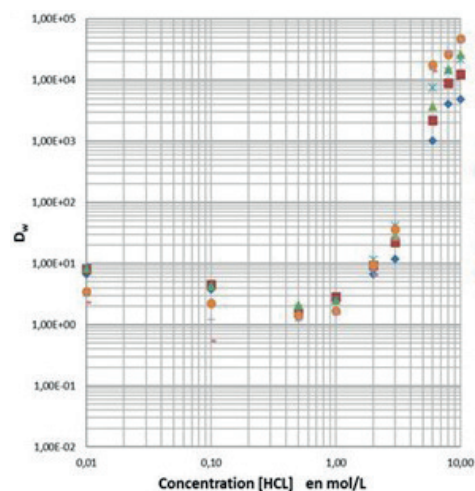
D_w values of selected elements on TK222 in HNO_3

None of the shown elements show significant retention on TK222 from HNO_3 .



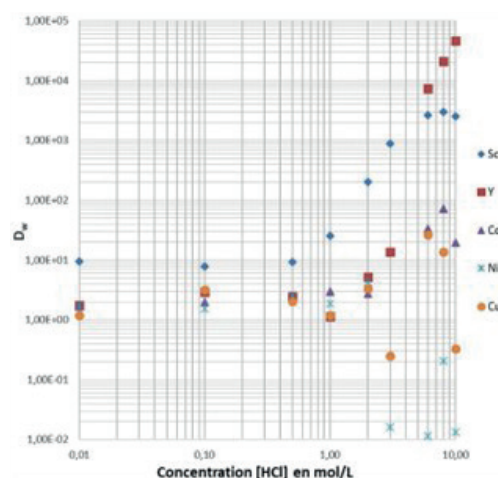
D_w values of selected elements on TK222 in HCl

Out of the shown elements only Tl is well retained at high HCl concentrations.



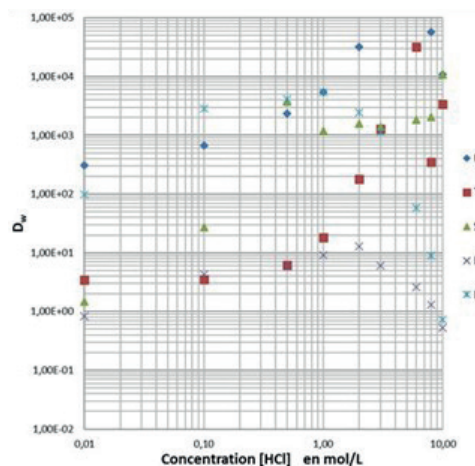
D_w values of selected elements on TK222 in HCl

Lanthanides are strongly retained at high HCl concentrations ($\geq 6\text{M}$) for example. As for HNO_3 this is an important information with respect to the separation of lanthanides from Ac.



D_w values of selected elements on TK222 in HCl

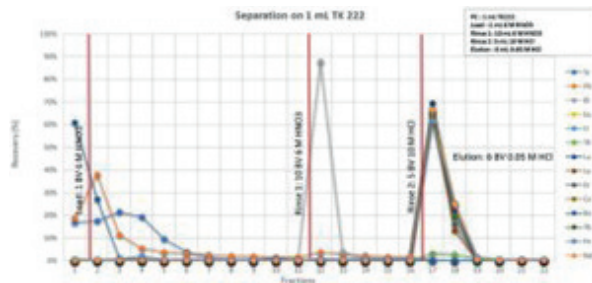
Like the Lanthanides Y and Sc are very well retained at high HCl concentrations. Co, Ni and Cu are not or only weakly retained.



D_w values of selected elements on TK222 in HCl



Under these conditions lanthanides, U and Th remain retained on TK222 Resin, while Ac is expected to elute which should result in a suitable separation of Ac from these elements.



Elution study, 2mL TK222 cartridge,
1 BV fractions, various elements.

When loading the TK222 Resin from 6M HNO_3 , followed by a rinse with the same acid, Pb, Ba and Sr are removed. Bi may then be removed using 10M HCl. As can be seen, under the usual Ac elution conditions (0.05M HCl) lanthanides would co-elute, accordingly they need to be removed as described before via the Ac elution from TK222 (or TK221) in very high HNO_3 .

Main applications

- Ac purification
- Lu-177 purification



TK21 1/2/3 Resins

The TK211, TK212 and TK213 Resins are based on different mixtures of organophosphoric, organophosphonic and organophosphinic acids. It could be shown that under certain conditions and for certain lanthanide pairs, such mixtures can show increased selectivity compared to the respective pure compounds.

The organic phase further contains a small amount of a long-chained alcohol that will act as radical scavenger to increase the radiolysis stability of the resin.

The inert support onto which the organic phase is impregnated contains aromatic groups which will also contribute to the increase of the radiolysis stability of the resins.

The inert support further shows an elevated capacity for the extractants. Accordingly, this allows the TK211/2/3 Resins to have a higher extractant load compared to e.g. the LN Resin series.

The TK211/2/3 Resins show, like the LN Resins, differences in their respective acidities. TK211 Resin is the most acidic resin, accordingly it will extract lanthanides, and other elements, at higher acid concentrations than e.g. TK212 and TK213 Resins. TK212 Resin on the other hand is more acidic than TK213 Resin (order of acidity: TK211 > TK212 > TK213).

The selectivity and retention of the lanthanides is generally very similar in HNO_3 and HCl on all three resins, accordingly both acids may be employed for the separation of lanthanides.

This difference in the relative acidity of the resins can be exploited to facilitate otherwise more complex lanthanide separations.

This will particularly be the case for the separation of very small amounts of one lanthanide from a large excess of its neighbouring lanthanide.

Typical examples are the production of nca Lu-177 (separation from irradiated Yb-176 targets) and nca Tb-161 (separation from irradiated Gd-160 targets).

By performing a first separation on a 'less acidic resin' such as TK212 Resin followed by direct elution of the lanthanide fraction to be further purified onto a more acidic resin such as TK211 Resin for further purification ("sequential separation") it is possible to eliminate intermediary steps such as the use of TK221 (or DGA) Resin to convert the lanthanide fraction from higher acid concentration to low acid concentration.

In an ideal case even a fully sequential three column separation might be possible (TK213 => TK212 => TK211).

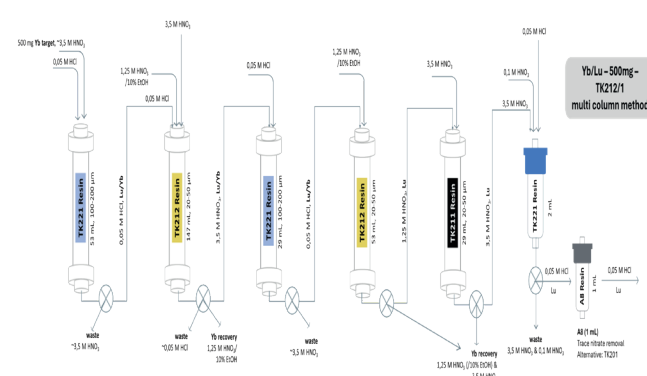
Two examples of the use of such sequential separation steps will be shown in the following.

The production of nca Lu-177 is rapidly gaining importance due to its increased use in nuclear medicine. Reliable, preferably easy to automatize methods that allow for its separation from irradiated Yb-176 targets of elevated size ($\geq 500\text{mg}$) are thus of increasing importance.

Horwitz et al. describe a method based on three LN2/DGA cycles for the separation of nca Lu-177 from 300mg Yb-176 targets. While this method gives good yields ($\sim 73\%$) in a short separation time ($\sim 4\text{h}$) the fact that an elevated number of columns are required complicates its automatization. Further it has only been tested for up to 300mg of target material.

By introducing a sequential separation step this method can be partially simplified.

It could be shown that the method described in the following figure allows for separating Lu from up to 500mg of Yb with elevated Lu recovery ($\sim 85\%$) and a very low amount of residual Yb in the final Lu fraction.

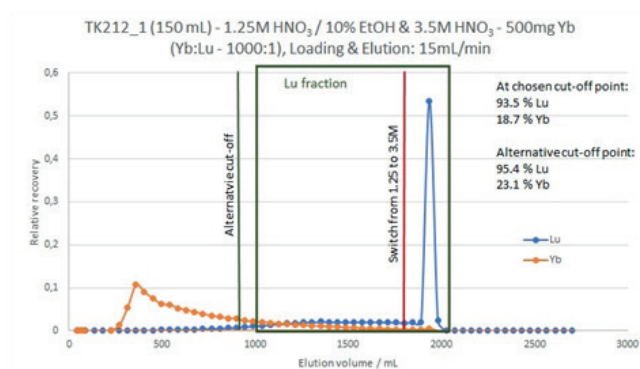


Scheme of a method for the separation of Lu from 500mg Yb using TK221, TK212 and TK211 Resins

The increased Lu recovery is, other than the use of TK212 Resin instead of LN2 Resin, also due to the adjustment of the eluting agent used for the chromatographic separation of Lu and Yb on the first TK212 Resin column from 1.3M HNO_3 , as suggested by Horwitz et al., to 1.25M HNO_3 / 10% EtOH.

It should be noted that adding EtOH only showed an improvement for 1.25M HNO_3 but not for 3.5M HNO_3 . Further, mixing 3.5M HNO_3 with EtOH should be strictly avoided for safety purposes.

The following figures show typical chromatograms obtained during the separation of Lu from 500mg of Yb (initial Lu:Yb ratio: 1:1000). All experiments were performed using stable elements, fractions of defined sizes were collected, diluted, and analysed off-line by ICP-MS. Relative recoveries were calculated for Lu and Yb and plotted against the elution volume.



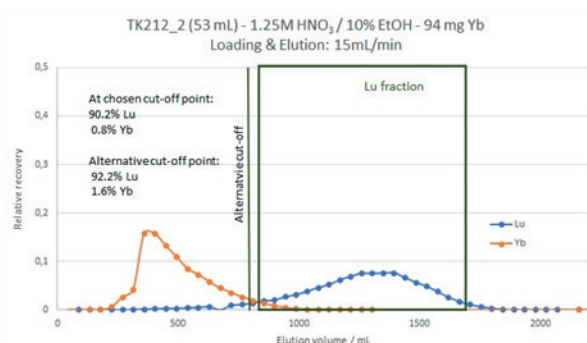
Example of a Lu separation from 500mg Yb on a TK212 Resin column (2.5 x 30cm, 150mL) using 1.25M HNO₃ / 10% EtOH and 3.5M HNO₃. Flowrate: 15mL/min.

It should be noted that the switch to 3.5M HNO₃ in the given example was made at a rather late stage of the separation. In the final version of the process it should take place earlier, ideally triggered via radiation detection, close to the chosen cut-off point (left end of the green frame).

Indeed, the moment of the switch will have, especially on the first column, a considerable influence on the Lu recovery and Yb carry-over. This is mainly due to the significant tailing introduced by the macro-amount of Yb.

The fractions comprised in the green frame ("Lu fractions") were combined and passed through a 5 - 10g TK221 Resin cartridge for conversion to ≤0.05M HCl. The Lu fraction thus obtained in dilute HCl was then loaded onto the next TK212 Resin column (1.5 x 30cm, 53mL).

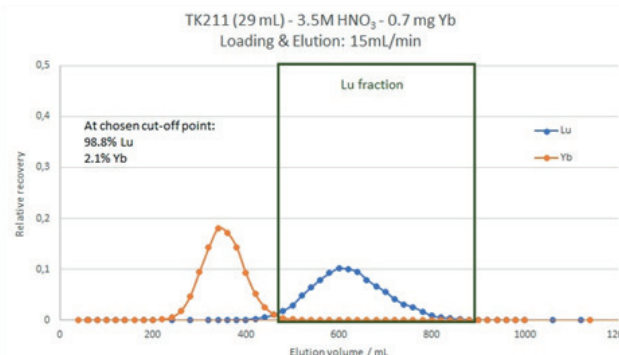
Due to the lower amount of Yb present on the column the tailing of the Yb and Lu elution is less pronounced than on the first TK212 column.



Example of a Lu separation from the Lu fraction obtained on the first TK212 Resin separation, containing 94mg Yb, on a second TK212 Resin column (1.5 x 30cm, 53mL) using 1.25M HNO₃ / 10% EtOH. Flowrate: 15mL/min.

Although this would be possible, in this example the Lu containing fractions (green frame) are not eluted in HNO₃ of elevated concentration (as described in the Horwitz method), passed through a TK221 (or DGA) Resin cartridge and eluted in dilute HCl for another load onto TK212 Resin.

Instead the combined fractions are directly loaded onto a TK211 Resin column (1.1 x 30cm, 29mL) for the final purification of the Lu.



Example of a Lu separation from the Lu fraction obtained on the second TK212 Resin separation, containing <1mg Yb, on a TK211 Resin column (1.1 x 30cm, 29mL) using 3.5M HNO₃. Flowrate: 15 mL/min.

Lu is finally obtained following separation/elution e.g. with 3.5M HNO₃.

As final step the obtained Lu fractions (as indicated in the green frame) were combined and loaded onto a 2mL TK221 Resin cartridge, any last potentially present metallic impurities are removed through consecutive rinses with 3.5M HNO₃ and 0.1M HNO₃. Lu is then finally eluted using ≤0.05M HCl.

Last traces of nitrates that might still be present will be removed via a 1mL anion exchange cartridge (A8 Resin).

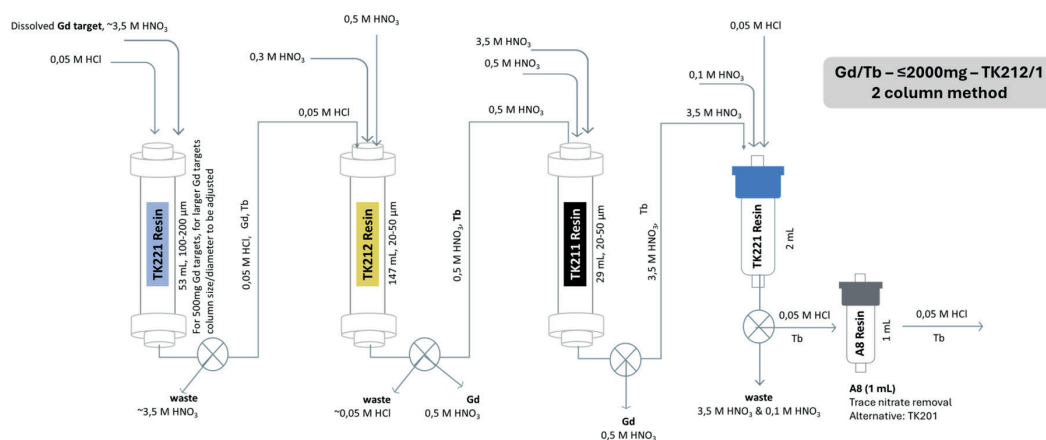
Further upscale of this separation process is currently being finalized.

Another radiolanthanide increasingly finding use is Terbium. As Tb isotopes may be used for PET imaging (Tb-152), SPECT imaging (Tb-155), alpha therapy (Tb-149) and beta therapy (Tb-161) it is also referred to as the 'swiss army knife' of nuclear medicine.

Particularly the interest in Tb-161 is currently increasing significantly, accordingly methods for the separation of Tb from irradiated Gd targets of elevated size are needed.

The development, and later upscale, of methods for the separation of Tb from 500-2000mg Gd is currently ongoing. The next figure shows a scheme of a suggested separation process.

As may be seen the separation is more straightforward compared to the separation of Lu from Yb targets.



Scheme of a method development for the separation of Tb from 500 - 2000mg Gd using TK221, TK212 and TK211 Resins. Note: the size of the first TK221 Resin columns needs to be adjusted..

The following two figures show typically obtained chromatograms (stable Gd, Tb and Dy, with an original ratio of 1000:1:1).

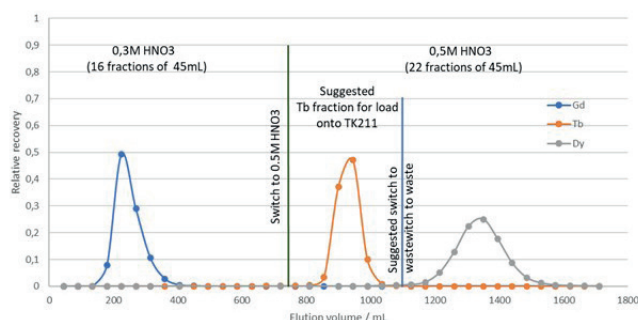
Like for the Lu separation, the Gd/Tb/Dy separations were performed using stable elements, fractions of defined volumes were taken and analysed by ICP-MS.

The first separation is performed on a TK212 Resin column. This step allows for the separation of Tb from Gd and Dy.

The obtained Tb fractions (indicated by the orange frame) were then combined and directly loaded onto a TK211 Resin column for final purification of the Tb (polishing).

As may be seen under the chosen conditions most of the Gd is breaking through during the load onto the TK211 Resin column, any Gd remaining on the column is then removed with 0.5M HNO₃.

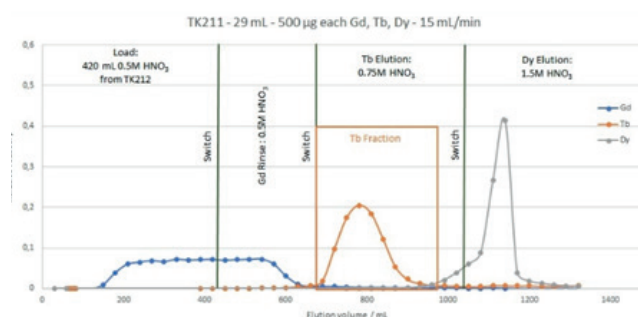
It could be shown for the Lu separation process that the addition of small amounts of EtOH (10% v/v) improves the separation, this is currently also being tested for the Tb separation.



Example of a Tb separation from 1000mg Gd on a TK212 Resin column (2.5 x 30cm, 147mL) using 0.3M HNO₃ and 0.5M HNO₃ Flowrate: 15mL/min

Increasing the concentration of the mineral acid (in this example to 0.75M HNO₃) will lead to the elution of Tb, leaving potentially remaining traces of Dy on the

columns. In case the presence of Dy can be ruled out this elution can be performed at higher acid concentrations (e.g. 3.5M HNO₃ for direct load onto a 2mL TK221 Resin cartridge), thus lowering the elution volume.



Example of a Tb separation from 500µg Gd on a TK211 Resin column (1.1 x 30cm, 29mL) using 0.5M HNO₃ and 0.75M HNO₃. Flowrate: 15mL/min.

As a final step the Tb will be concentrated on a 2mL TK221 Resin cartridge, any last potentially present metallic impurities are removed through consecutive rinsed with 0.75M HNO₃ and 0.1M HNO₃. Tb is the finally eluted using ≤0.05M HCl.

Last traces of nitrates that might still be present will be removed via a 1mL anion exchange cartridge (A8 Resin).

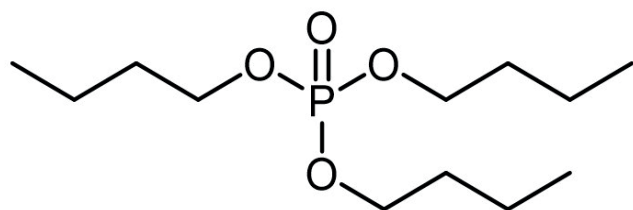
The indicated method is currently undergoing further optimisation and upscale.

Prepacked TK211/212/213 Resin columns of various sizes (e.g. 375mL, 147mL, 53mL and 29mL) are now commercially available.

Main applications

- Lanthanide separation e.g. n.c.a Lu-177 and n.c.a Tb-161

TBP Resin



TriButylPhosphate (TBP)

The TBP Resin is comprised of an inert support impregnated with Tributylphosphate (TBP). TBP is a widely used extractant, it finds e.g. application in the Purex process, the reprocessing of U and Pu from spent fuel. Other applications include, amongst others, the separation of yttrium for analytical purpose.

The TBP Resin has been characterized with respect to the uptake of various elements in HNO_3 and HCl .

Beside Pu(IV) and Np(IV) several other elements such as Au, Zr, Hf, Fe, Sn and Ga also show high affinity for the TBP Resin in HCl .

While Au remains retained under all tested conditions, making its elution rather difficult in HCl , the other elements only show high D_w values at elevated acid concentrations, and low D_w values at lower concentrations.

In 1M HCl for example only Sn shows elevated D_w values whereas Fe, Ga, Sb, etc. show very little affinity to the resin, allowing for its separation from these elements. Sn can then be eluted e.g. with 0.1M HCl .

The TBP Resin generally shows very good selectivity for Sn over Te (Te-126 is an isobaric interference for the mass spectrometric determination of a long-lived beta emitter Sn-126, frequently determined in decommissioning and radioactive waste samples) and over Cd which is frequently used as target material for the production of Sn-117m, a conversion electron emitter used in nuclear medicine.

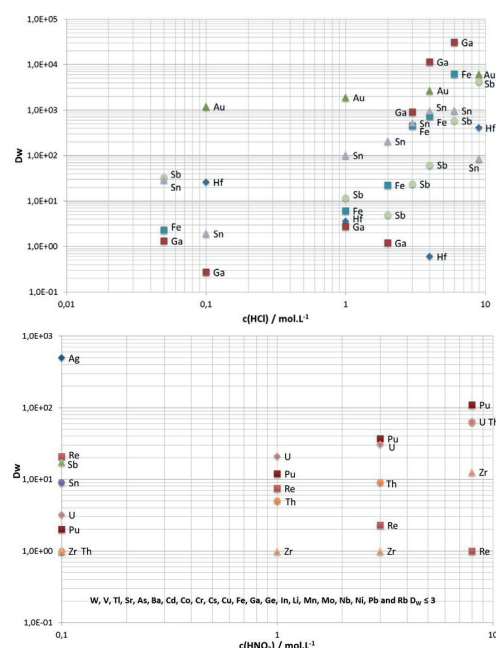
The resin also shows interesting selectivity for Sb, however its oxidation state needs to be carefully controlled.

It has further been employed by Graves et al. for the separation of Zr-89 from Y targets. Lyashchenko et al. further optimized the Zr-89 separation on TBP Resin by using two TK400 cartridges upfront, mainly to remove Fe. A radionuclidic purity of >99% and Fe, and other metallic impurities, content in the final product of <1ppm could be achieved. The obtained Zr-89 was used to successfully label PSMA-617 and PSMA-I&T. It is also used for the separation of Sc from Ca targets. In both cases the products were retained from high HCl and eluted in dilute HCl . Under similar conditions a Ga separation from Zn should be feasible.

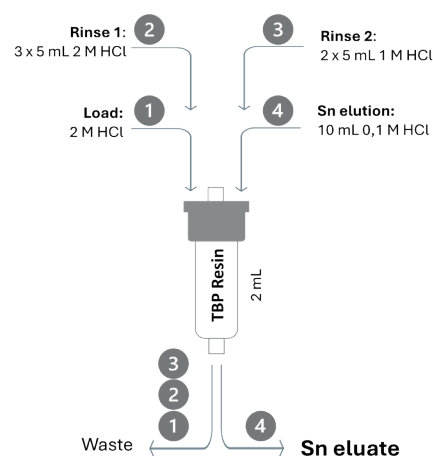
In HNO_3 of all tested elements only the actinides (at elevated HNO_3 concentrations), and Ag (at low HNO_3 concentrations), are retained on the resin.

Based on the obtained data, Dirks et al. have developed a procedure for the separation of Sn from various elements.

Most of the tested elements are not retained during load and first rinse. Sn and part of the Ga and Fe are retained. The latter two are first removed with 1M HCl , Sn is then quantitatively eluted with 6mL 0.1M HCl in high purity.



D_w values of selected elements on TBP Resin in HCl and HNO_3



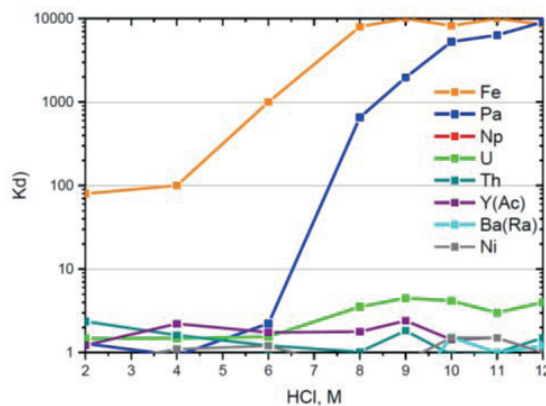
Separation scheme Sn separation on TBP Resin

Main applications

- Separation of zirconium
- Separation of tin
- Separation of scandium
- Separation of actinides

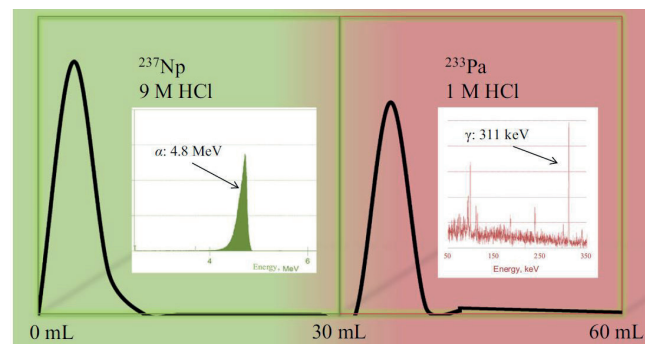
TK400 Resin

The TK400 Resin is an extraction chromatographic resin that is comprised of a long-chained alcohol impregnated onto an inert support. Knight et al. showed that long-chained alcohols, especially octanol, show very interesting selectivity towards Pa at high HCl concentrations, allowing for facile Pa/Np separation using column chromatography. Jerome et al. characterized the TK400 Resin with respect to its selectivity for a number of elements including Pa, Np, U and Th.



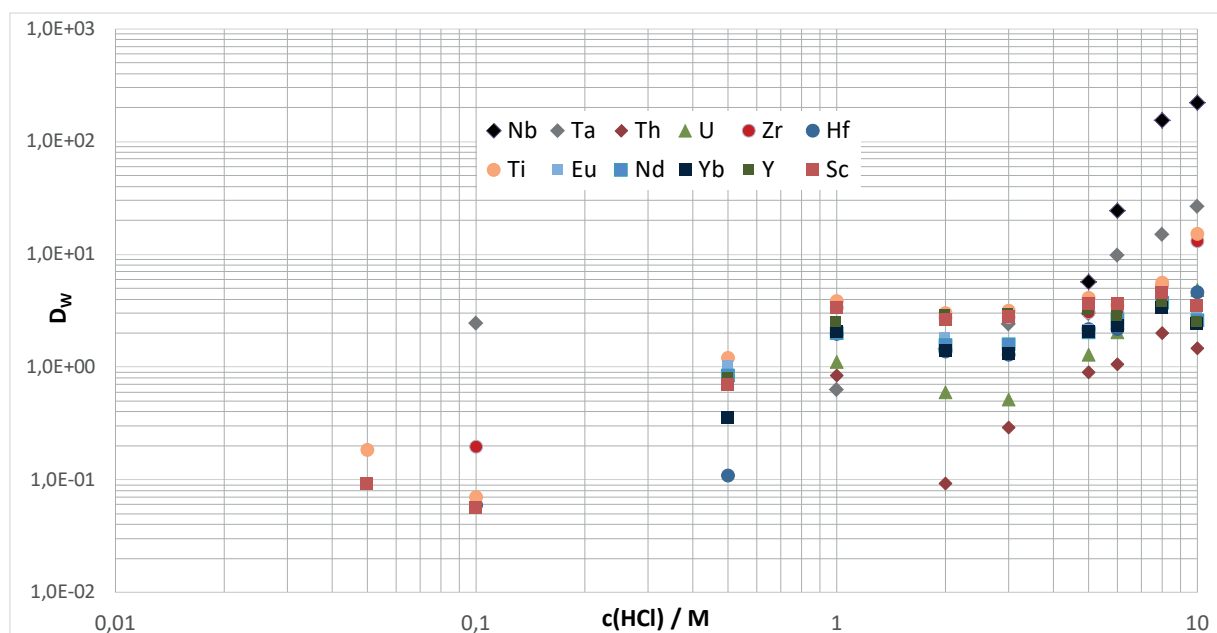
D_w values of selected elements on TK400 Resin in HCl at varying concentration [Data provided by Ivanov et al]

They found that Pa retention sharply increases at high (≥ 9 M) HCl concentrations whereas other elements tested are not retained. At HCl concentrations < 8 M HCl on the other hand D_w values of Pa were found to be low allowing for its elution in a small volume. Ostapenko et al. found a similar trend for Pa retention with k' values being high for Pa at high HCl concentrations (9 M). These results correspond overall well to the selectivity observed by Knight et al. when performing Np/Pa separation.

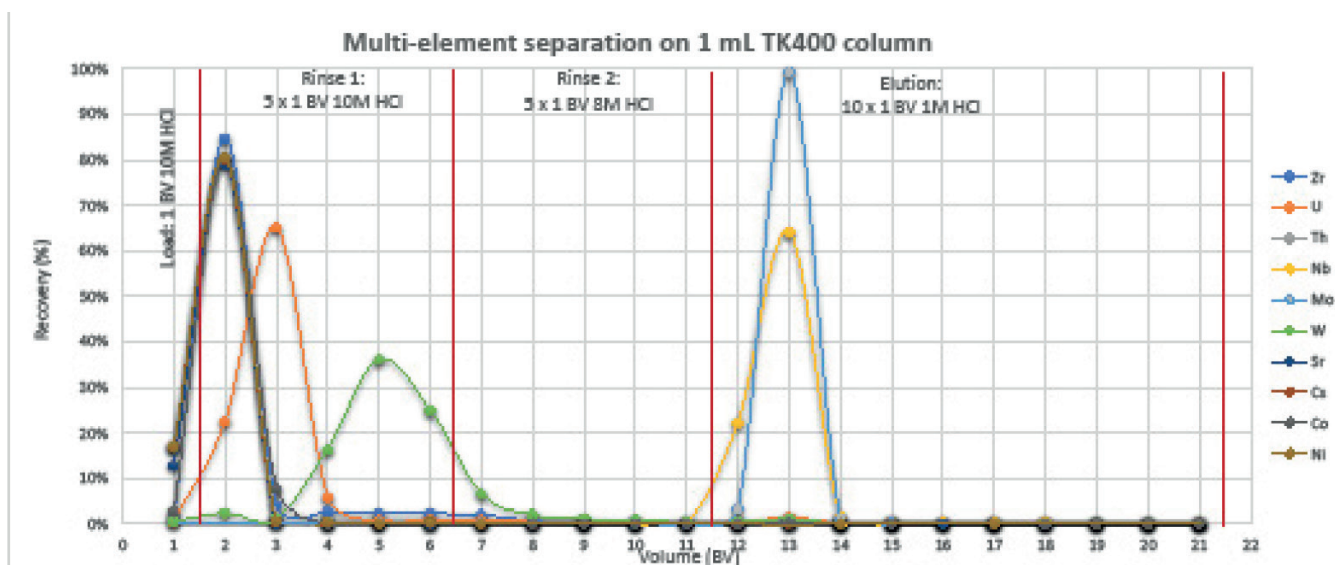


Elution study, Np/Pa separation on long-chained alcohol resin [Taken from Knight et al.]

The resin shows high selectivity for Nb at high HCl concentrations over other elements tested such as Ta, Zr, Hf and lanthanides which are not, or only very poorly as in the case of Ta, retained by the resin.

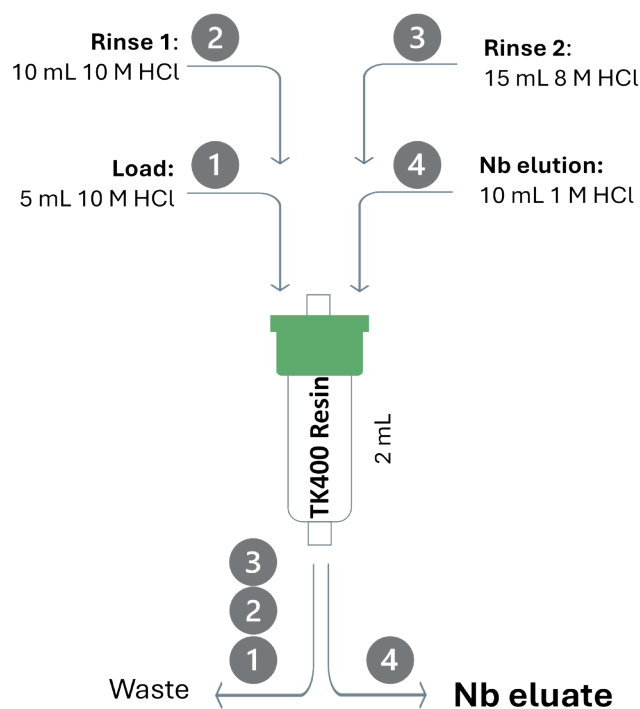


D_w values of selected elements on TK400 Resin in HCl at varying concentration



Elution study, Nb separation from selected cations, 2mL TK400 Resin column

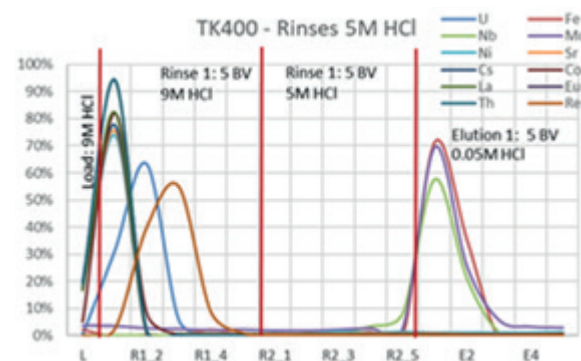
With respect to its selectivity the TK400 Resin shows the potential for allowing a number of interesting separations such as Nb/Zr (see Nb separation scheme below) and Pa/U/Th. The results of an elution study on the separation of Nb from a number of elements, including Zr, and the separation method used to obtain these results are shown in following figures. Jerome et al. employed the TK400 Resin for the separation of Pa from its descendants. They found that U, Th, Ac, Ra and Pb were removed from the resin during load and rinse, allowing for obtaining a clean Pa fraction with high chemical yield (~83%).



Nb separation on TK400 Resin

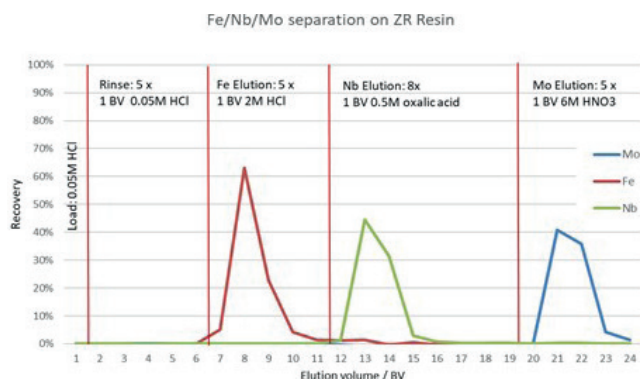
Another interesting application of the TK400 Resin was described by Tieu et al. and Svedjehed et al. Both authors used the TK400 Resin for the separation of Ga-68 from irradiated solid Zn targets. Svedjehed et al. showed that it is advantageous for solid targets to use the TK400 Resin instead of ZR Resin in combination with A8 Resin and TK200 Resin.

The fact that the TK400 Resin shows higher Fe capacity than e.g. TRU Resin makes its use in the analysis of i.e. decommissioning samples interesting. A method combining the use of the TK400 Resin (separation of Fe, Nb and Mo from most matrix elements) and ZR Resin (subsequent separation of Fe from Nb and Mo) is currently being optimized. The following graphs show typically obtained elution profiles.



Fe/Nb/Mo separation from selected elements, TK400 Resin

As may be seen, while Fe, Nb and Mo are well retained a large number of other elements, such as e.g. Zr, U, Th, Cs, Co,... are removed during load and rinse. These three elements may then be eluted in dilute HCl and directly loaded onto ZR Resin for further separation.



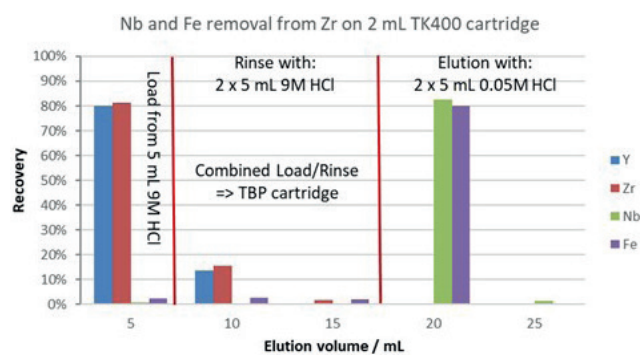
Fe, Nb and Mo separation on ZR Resin

The fact that the TK400 Resin shows high selectivity for Fe and Nb but not for Zr may also allow its use in the separation of Zr-89 from solid Y targets, with the aim to further lowering the amount of these impurities in the final product.

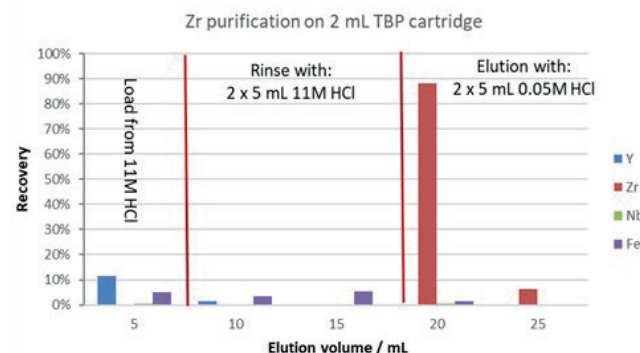
First tests showed that loading a simulated dissolved target solution containing Zr, Y, Nb and Fe through a TK400 Resin cartridge at 9M (or 10M) HCl, followed by a rinse under the same conditions will allow retaining Nb and Fe on the TK400 Resin while Zr (and Y) will pass through.

Combining the load and rinsing fractions containing Zr, adjusting them to 11M HCl and loading this solution through a TBP Resin cartridge (similar to the method described by Graves et al.) will then allow a clean Zr separation with high chemical yield. It should be noted that alternatively 10M HCl may be used as loading condition on both cartridges. This could simplify the separation, as the intermediate HCl concentration adjustment is not necessary. It might further allow for the use of stacked cartridges in the initial loading step, this modification will require further testing though. Zr is finally eluted from the TBP Resin cartridge using dilute HCl.

If desired, Nb and Fe may be recovered from the TK400 Resin cartridge using dilute HCl. The following graphs show typically obtained elution studies under the described conditions.



Nb and Fe removal from Zr (and Y) on a 2mL TK400 Resin cartridge



Zr purification on a 2mL TBP Resin cartridge

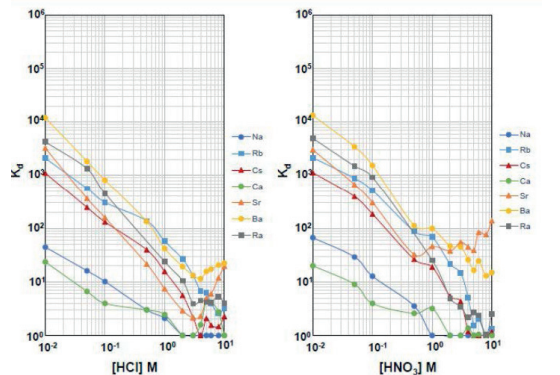
Main applications

- Separation of zirconium in combination with TBP Resin
- Separation of tin
- Separation of scandium
- Separation of actinides



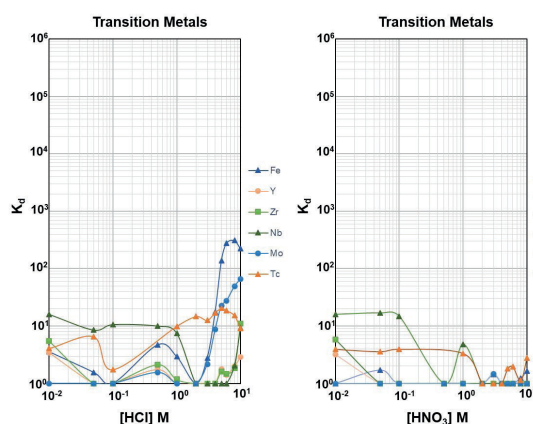
TK101 Resin

The TK101 Resin is based on the same crown-ether that is also used in the SR, PB and TK102 Resin. As diluent an ionic liquid is employed, this leads to a quite significant change in its selectivity at low acid concentrations, as can be seen in the following graphs.



D_w values of selected elements on TK101 Resin in HNO_3 .
Data courtesy of Ben Russel (NPL)

While SR, PB and TK102 Resins show no selectivity for Sr, Ba, Ra at low acid concentrations TK101 Resin does. In dilute HNO_3 as well as in HCl Ba, Ra and Sr are very well retained from $\leq 0.01\text{M}$ to approx. 0.05M . Their retention decreases significantly as the acid concentration increases though. Sr retention in HNO_3 is an exception as it shows increasing retention at HNO_3 concentrations $> 3\text{M}$. It should be noted that, while Ra and Ba behave similar at lower acid concentrations, they do show differences at higher acid concentrations, notably at 3M HNO_3 , where the k_D Ba is higher than k_D Ra, thus allowing for Ra/Ba separation.

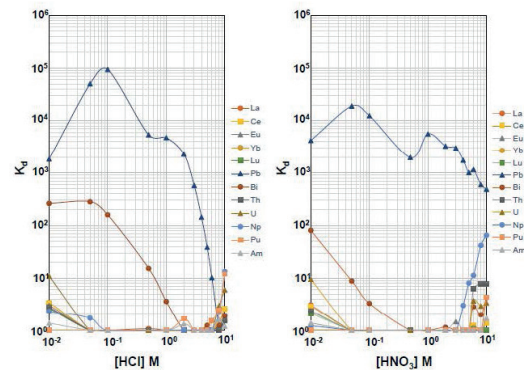


D_w values of selected elements on TK101 Resin in HCl and HNO_3 . Data courtesy of Ben Russel (NPL)

Pb is very well retained over the whole HNO_3 concentration range. It is also very well retained from HCl up until approx. 3M HCl. Its retention then decreases very sharply as the HCl increases, making HCl of high concentration suitable for Pb elution. Out of the elements shown in the following graph only Bi is also retained, particularly from dilute HCl to elevated acid concentrations. It can be very easily

removed though, especially using HNO_3 . Lanthanides and actinides are not retained on TK101 not in HCl neither in HNO_3 . Pb is very well retained over the whole HNO_3 concentration range. It is also very well retained from HCl up until approx. 3M HCl. Its retention then decreases very sharply as the HCl increases, making HCl of high concentration suitable for Pb elution.

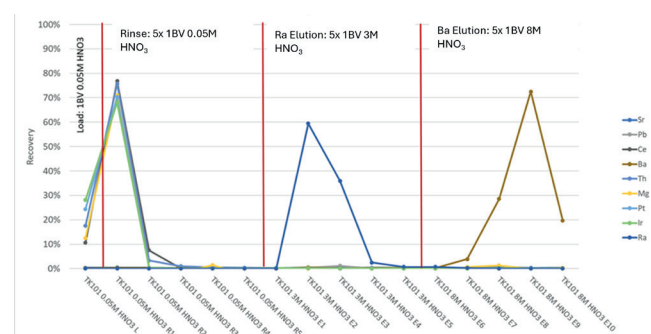
Out of the elements shown in the graph above only Bi is also retained, particularly from dilute HCl at elevated acid concentrations. It can be very easily removed though, especially using HNO_3 . Lanthanides and actinides are not retained on TK101 nor in HCl neither in HNO_3 .



D_w values of selected elements on TK101 Resin in HNO_3 .
Data courtesy of Ben Russel (NPL)

Out of the tested transition metals only Nb shows some retention at high HCl, all other elements tested show very low k_D values.

The graph below shows the result of an elution study performed with amongst others Th, Ce, Pt, Ir, Ba, Ra and Pb.



Elution study of selected elements on TK101 Resin
Ra data courtesy of Nora Vajda (RadAnal)

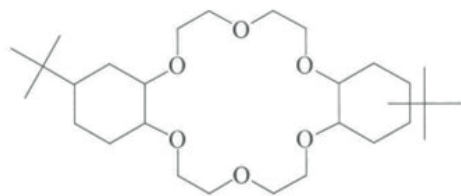
Other elements tested than Ba, Ra and Pb are removed during load and rinse with dilute acid (here 0.05M HNO_3). Ra is then eluted with 3M HNO_3 , while Ba and Pb remain retained. Ba is eluted with 8M HNO_3 . Pb is still retained under these conditions and may then be eluted with $6-8\text{M}$ HCl or citrate/citric acid.

Main applications

- Ra separation
- Pb separation.

TK102 Resin

The TK102 Resin is based on the same crown-ether that is also used in the SR and PB Resin.

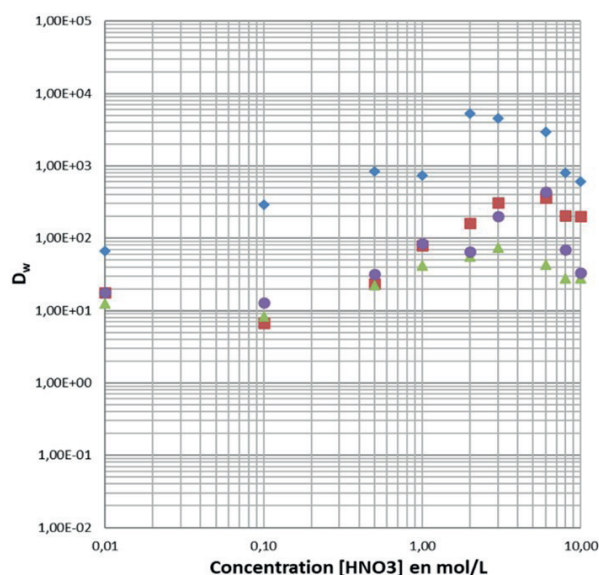


4,4'(5')-di-t-butylcyclohexano-18-crown-6

Other than these resins the TK102 Resin contains a long-chained fluorinated alcohol as diluent. The resin further contains a larger amount of the crown-ether compared e.g. to the SR Resin. Further the organic phase is impregnated onto an inert support containing aromatic groups for increased stability against radiolysis. The resin was originally optimized for the separation of Ba and Ra, however it also shows very interesting properties with respect to Sr and Pb separation.

The following two graphs show the selectivity of the TK102 Resin for a range of elements in HNO_3 and HCl. The third graph shows the influence of increasing amounts of Na, K and Ca on the Sr retention in 3M HNO_3 .

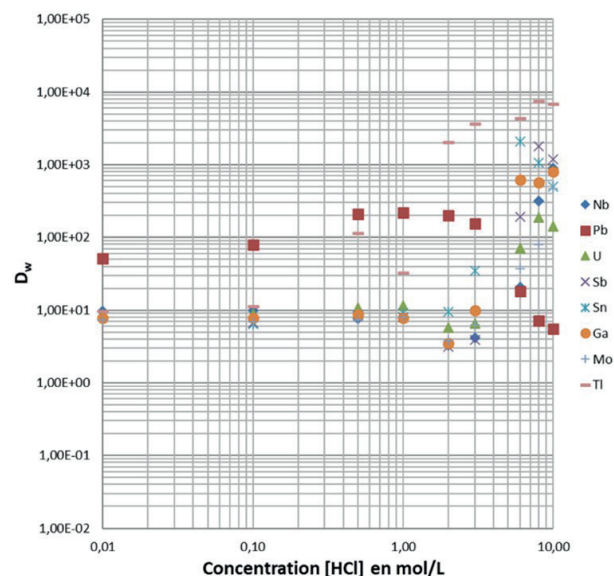
All D_w shown in these graphs were obtained through ICP-MS measurements.



D_w values of selected elements on TK102 Resin in HNO_3

Pb is very well retained over the whole HNO_3 concentration range. Sr is well retained at elevated HNO_3 concentrations (3 – 10M HNO_3), showing higher Sr D_w values than the SR Resin under these conditions. The same is true for Ba at 3M HNO_3 , TK102 shows stronger Ba retention than SR Resin.

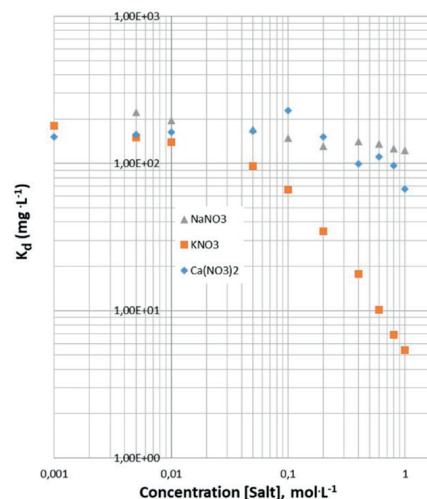
Further it is notable that Tl is strongly retained from 3 – 6M HNO_3 .



D_w values of selected elements on TK102 Resin in HCl

As expected, Pb is well retained over a wide HCl concentration range, from dilute HCl up to 2 – 3M HCl. Pb D_w values drop strongly for higher HCl concentrations ($\geq 6\text{M}$ HCl), allowing for its elution under these conditions.

The TK102 Resin retains, to a certain extent similar to the TK400 Resin, a number of elements at very elevated HCl concentrations, including Tl, Sb, Sn, Ga and Nb.



D_w values Sr on TK102 Resin in 3M HNO_3 and in presence of increasing amounts of Na, K and Ca

Na shows very little influence on the Sr retention on the TK102 Resin, even at concentrations up to 1M D_w values for Sr remain high.

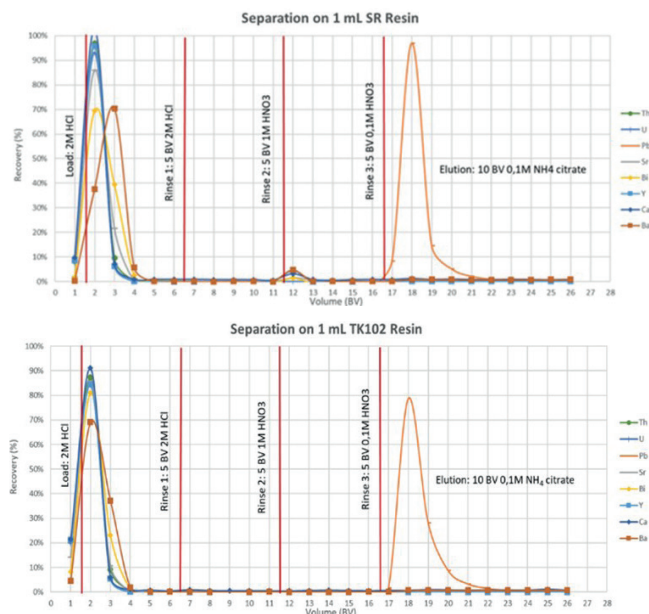
Ca is showing a higher impact, nevertheless even at concentrations up to 0.5M Sr shows elevated D_w values. As expected, K is interfering with the Sr retention very strongly, even concentrations $\geq 0.05\text{M}$ will lead to a significant decrease in Sr retention.

Just like for the SR Resin, performing a co-precipitation (e.g. with calcium phosphate) to remove K before the actual separation on TK102 Resin is crucial.

The following figures are showing three comparative elution studies on TK102 Resin and SR Resin.

The first example is a typical Pb separation based on loading from 2M HCl, Po removal with dilute HNO₃ and finally Pb elution with citrate.

Both resins are showing very similar elution profiles, TK102 Resin might require a slightly larger elution volume for Pb though. Nevertheless, typically employed elution volumes (e.g. 10mL) should assure quantitative elution of Pb also from TK102 Resin.

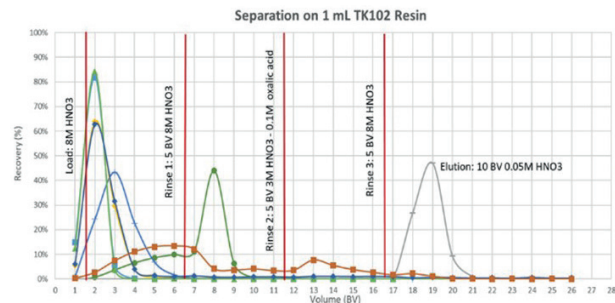
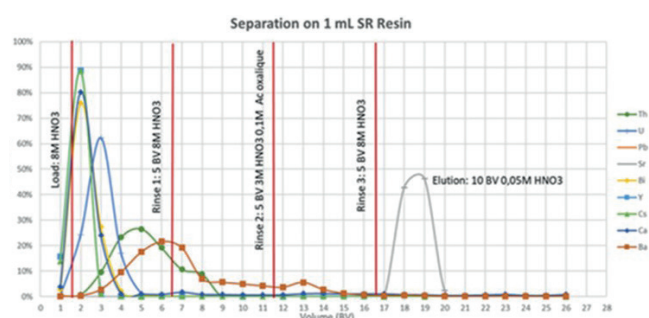


Comparative elution studies,
SR and TK102 Resin, Pb separation

The second example is a typical Sr separation based on loading from 3M HNO₃, rinsing with 8M HNO₃ and 3M HNO₃/0.1M oxalic acid, and finally Sr elution in 0.05M HNO₃.

Again, both resins are showing similar elution profiles. One distinct difference being Th, for the TK102 Resin 3M HNO₃/0.1M oxalic acid rinse is required to remove most of the Th while on SR Resin the majority is already removed with 8M HNO₃.

Like for the Pb separation Sr elution from TK102 Resin seems to require slightly larger volumes, but here too typically employed elution volumes (10 – 15mL) seem to assure quantitative Sr elution.

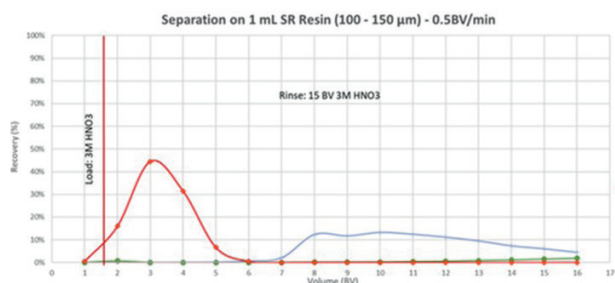


Comparative elution studies,
SR and TK102 Resin, Sr separation

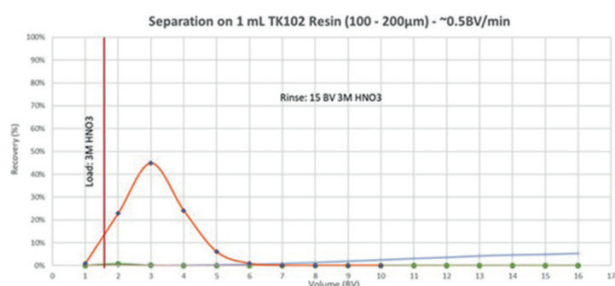
The third example shows a comparative Ba/Ra separation elution study. TK102 and SR Resin were both loaded from 3M HNO₃, then both resins were rinsed with several bed volumes (BV) of 3M HNO₃.

For both resins Ra is eluted quickly during load and first rinsing steps, while Ba remains retained.

On the SR Resin Ba starts to significantly break through after 6 BV, on the TK102 Resin the Ba retention is distinctively stronger, it starts to very slowly elute after about 8 - 9 BV.



Comparative elution studies, SR and TK102 Resin,
Ba/Ra separation



Further the TK102 Resin shows high dynamic capacity for Sr (>40mg/g) and Pb (>90mg/g).

Due to the higher hydrophobicity of the diluent employed in the TK102 Resin it also shows significantly (>10 times) less bleeding of organic material, measured as Non-Purgeable Organic Carbon (NPOC), than the SR Resin.

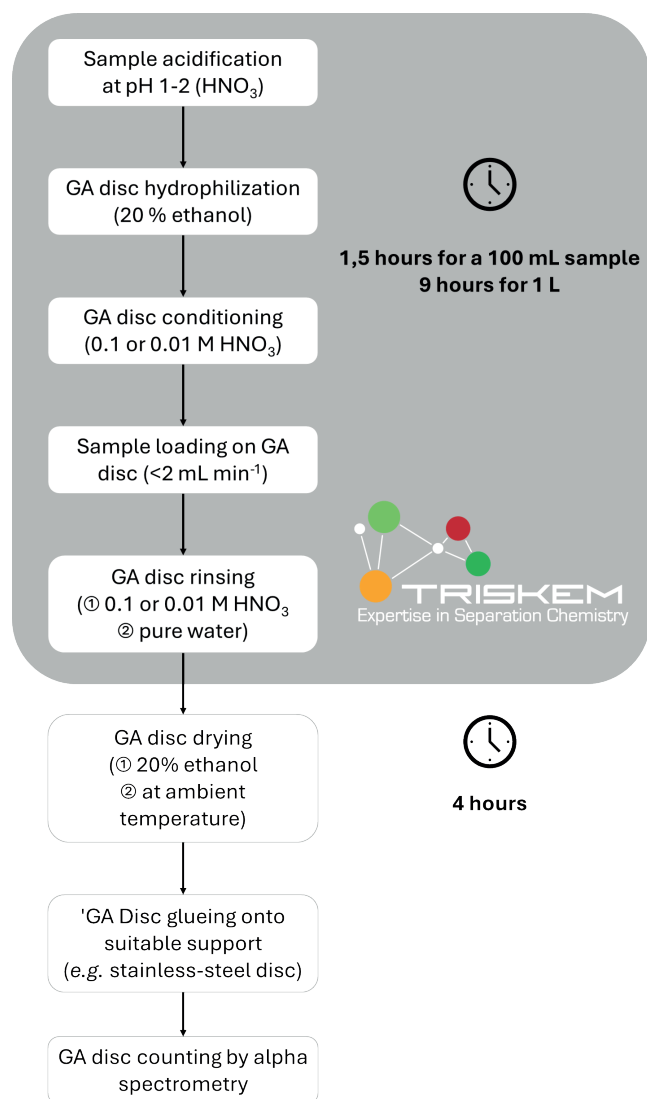
Main applications

- Ra/Ba separation
- Sr separation
- Pb separation

TK-GA Discs

The TK-GA (Gross Actinides) Discs are specialty membrane filters impregnated with an extractant system that shows very strong retention of actinides from acidified (pH 1 or 2, for some actinides even up to 3M HNO_3) water samples. They are the first product of a new range of such impregnated membrane filters ('Discs'). It's main application is the screening for actinides in aqueous samples via alpha spectrometry [Bailly et al.]. Other potential applications of such Discs include passive sampling [Wagner et al.] or analyte preconcentration via filtration [Horstmann et al.].

An outline for a typical actinide screening procedure in aqueous samples is shown below.



Sample preparation scheme TK-GA Discs for alpha spectrometry

The aqueous sample is first acidified to pH 1 or 2 and, in case this is necessary, filtered to remove fine particles that could interfere with the alpha spectrometric measurement. The TK-GA Disc is then preconditioned with 20% EtOH (hydrophilization step) followed by rinses with water and 0.01M or 0.1M HNO_3 (for Th e.g. it is preferable to work at pH 1).

The aqueous sample is then filtered through the TK-GA Disc, for best recovery and peak resolution of the alpha spectrum the filtration should ideally be performed at $\leq 2\text{ mL/min}$.

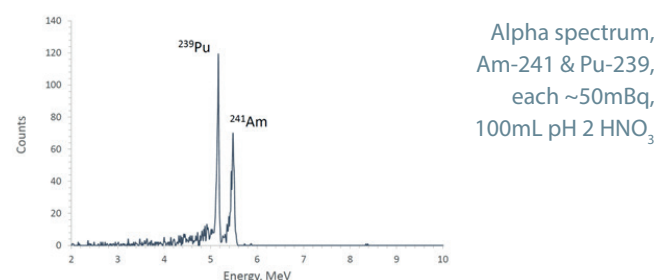
It should be noted that the TK-GA Discs are available in two diameters: 25mm and 47mm. 25mm discs are typically used for up to ~100mL samples, the 47mm discs for up to 1L samples.

Once the sample has been completely passed through the Disc it is subsequently rinsed with dilute acid, water and 20% EtOH, and air dried.

The dried disc is then glued onto a suitable support (e.g. stainless steel disc), and is then introduced into an alpha spectrometry system for measurement.

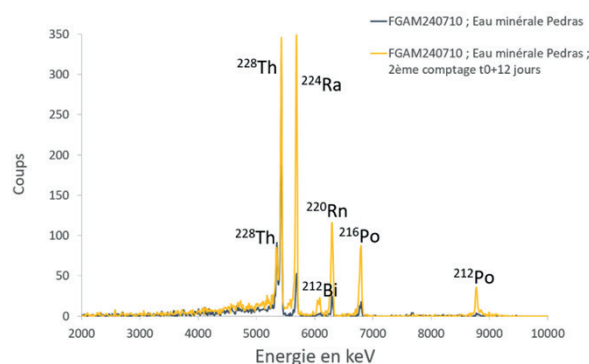
The following graphs show some examples of alpha spectra obtained using the TK-GA Discs.

All shown spectra were obtained by Bailly et al. as part of the LabCom 'TESMARAC' cooperation.



As can be seen, the spectral resolution of the Am and Pu peaks are good to very good (30 – 100keV) for a filtration based sample preparation. Typically 80 - >90% of the respective actinides are detected by alpha spectrometry, indicating that the actinides are to a very high extent fixed on the surface of the Discs.

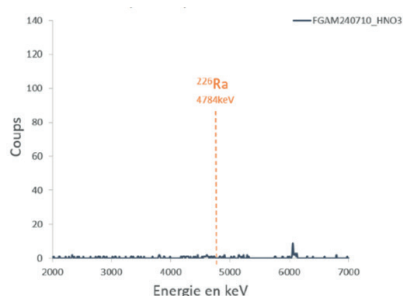
The following fig. shows that Th is very well retained from acidified Pedras mineral water (acidified to ~0.1M HNO_3), Ra on the other hand is not retained, and Po only weakly. The third figure further shows the alpha spectrum of Pedras water obtained directly after filtration, and after 12 days of ingrowth of its daughters.



Alpha spectrum, 47mm TK-GA Disc,
Pedras mineral water, 100mL pH 1.25

The fig. below shows the alpha spectrum of a Ra-226 containing solution at 0.01M HNO₃.

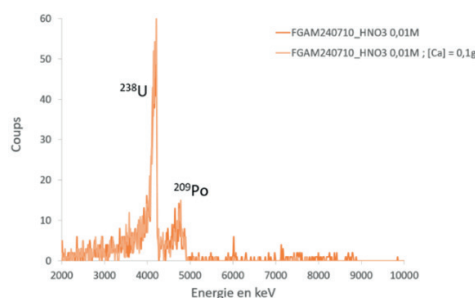
It could be confirmed that Ra-226 is not showing any signal in the alpha spectrum, it is indeed not retained as indicated in the Pedras water test. In an additional experiment at pH 4 it could be shown that while Ra is retained on the disc at this pH again no signal could be detected in the alpha spectrum, indicating that Ra is retained within the disc, and not on the surface.



Alpha spectrum, 47mm TK-GA Disc,
100mL Ra-226 containing solution, 0.01M HNO₃

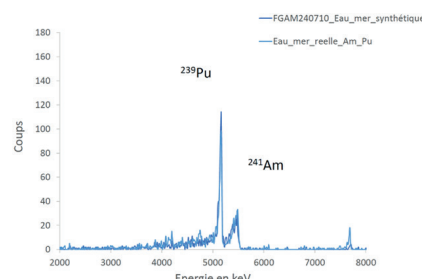
A similar test was performed with a solution containing U-238 and Po-209 in 0.01M HNO₃. While U is very well retained, Po is only showing partial retention.

Here, the influence of Ca especially on the U retention was tested by running the test also at elevated Ca concentrations (0.1g/L), no adverse effect was found to be induced by Ca.

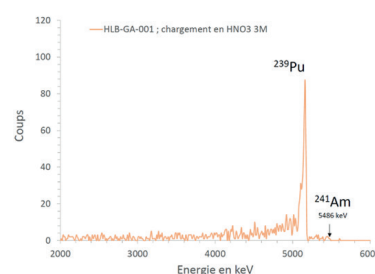


Alpha spectra, 47mm TK-GA Disc,
100mL U-238 and Po-209 containing solutions, 0.01M HNO₃

Pu is even strongly retained from high matrix samples such as sea water and highly acidic samples such as 3M HNO₃. Under the same conditions Am is distinctively less well (sea water) or not retained (3M HNO₃).

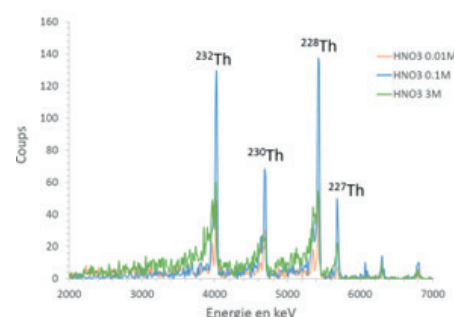


Alpha spectrum, 47mm TK-GA Disc, 100mL natural and
synthetic sea water samples containing Am and Pu,
acidified to pH 2



Alpha spectrum, 47mm TK-GA Disc, 100mL solution containing
Am and Pu, acidified to 3M HNO₃

Th is also retained from 3M HNO₃, not quite as well as from pH 1 or 2 though, and the peak resolution is distinctively less sharp.



Comparison of alpha spectra obtained with 47mm TK-GA Discs
for Th solutions of varying acid concentrations
(0.01M, 0.1M and 3M HNO₃)

Main applications

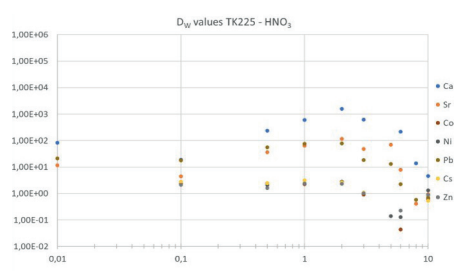
- Screening/alpha spectrometry measurement of actinides

TK225 Resin

The TK225 Resin is based on a mixture of a diglycolamide and an ionic liquid. The organic phase is impregnated onto an inert support containing aromatic groups for increased stability against radiolysis.

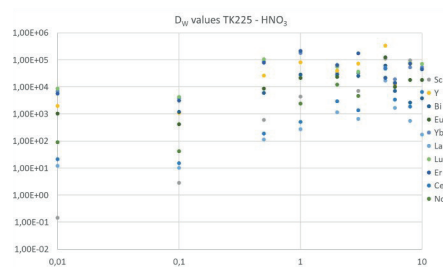
The main application of the TK225 Resin is the removal of radiolanthanides from acidic solutions, particularly from solutions of elevated HNO_3 concentration, for decontamination purposes.

The following graphs show the selectivity of the TK225 Resin for a wide range of elements in HNO_3 and HCl . All D_w shown in these graphs were obtained through ICP-MS measurements.



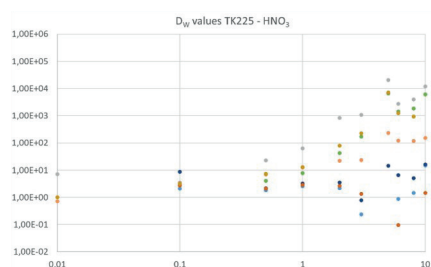
D_w values of selected elements on TK225 Resin in HNO_3

Out of the tested elements only Ca is quite strongly retained at elevated HNO_3 concentrations. Sr and Pb, too are retained under these conditions to a lesser extent though.



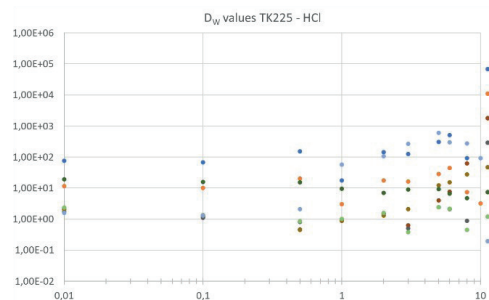
D_w values of selected elements on TK225 Resin in HNO_3

Lanthanides, especially heavy lanthanides, Y and Sc are very strongly retained from HNO_3 of elevated concentration. Especially for heavy lanthanides the D_w values remain very high, even at low HNO_3 concentrations.



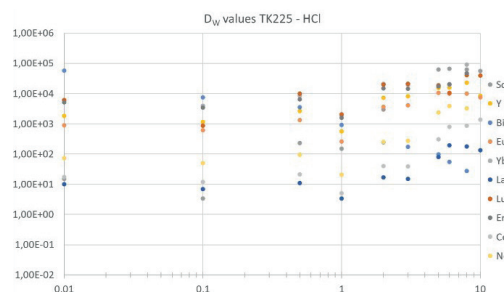
D_w values of selected elements on TK225 Resin in HNO_3

The TK225 Resin generally retains tetravalent elements such as Zr, Hf and Th at elevated HNO_3 concentrations quite strongly.



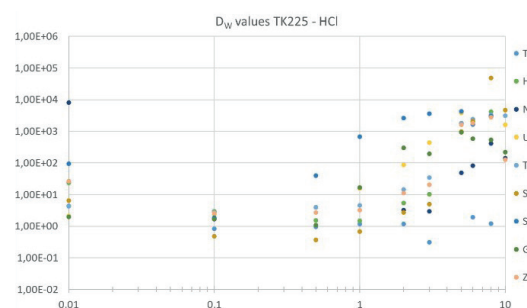
D_w values of selected elements on TK225 Resin in HCl

The TK225 Resin shows elevated retention of Ca and Zn at high HCl concentrations. Other elements shown are not or only very weakly retained.



D_w values of selected elements on TK225 Resin in HCl

Especially heavy lanthanides are well retained over a broad HCl concentration range, with highest retention being observed at high HCl concentrations. At high HCl concentrations Y, Sc and lighter lanthanides are well retained, too.



D_w values of selected elements on TK225 Resin in HCl

Higher valent elements such as Sb, Sn, Zr and U are well retained at high HCl concentrations, while showing very little retention at low HCl concentrations. TK225 Resin is mainly used for the removal of radiolanthanides, especially heavy radiolanthanides such as Lu-177, Yb-175, Tb-161,... from acidic solutions.

Especially the heavy lanthanides are near impossible to elute, accordingly the resin is mainly suitable for the decontamination of acidic effluents and waste solutions.

Main applications

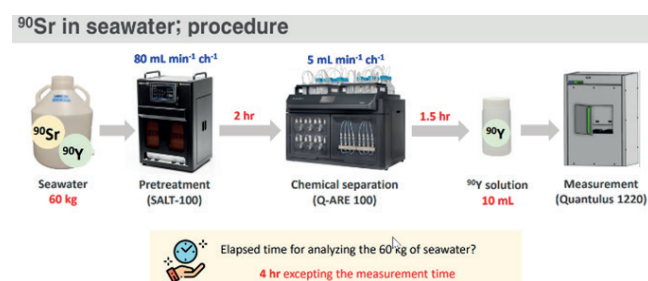


- Lanthanides removal from acidic effluents

TK227 Resin

The TK227 Resin is based on a diglycolamide (TO-DGA), small amounts of an ionic liquid and a long-chained alcohol. The ionic liquid is added to increase Y retention at lower HNO_3 concentrations. Its main application is the determination of Sr-90 in seawater samples via Y-90 separation and measurement as described by Kim et al.. The published method is a rapid procedure for the extraction and purification of Y-90 from large sea water samples (e.g. 60kg) that allows obtaining results in a few days (including measurement, the actual separation takes less than 4h) instead of several weeks. It thus represents a potential gain of time that very significant.

The TK227 Resin showed better performance than the originally employed DGA, N Resin and can thus replace the latter in the above-mentioned method. An overview over the method can be found below.



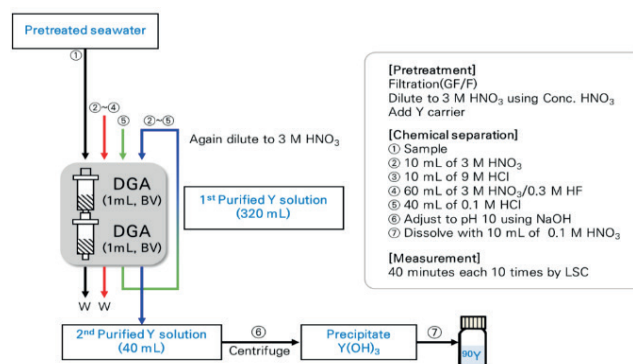
Workflow Sr-90 determination in seawater according to Kim et al.

Typically, 60kg seawater are pretreated (filtration, acidification to 3M HNO_3 (2M HNO_3 for TK227 Resin), followed by the addition of 3 – 4mg Y carrier.

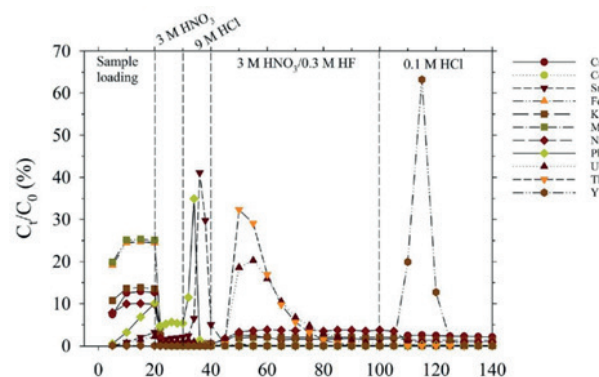
Y is preconcentrated from sea water samples on an equipment specifically developed for this type of applications (SALT-100, WITHEC Ltd, South Korea) at 80mL/min per channel using a combination of two 1mL DGA or TK227 Resin cartridges per channel (overall 16 cartridges). With eight channels being used the overall flow rate is in the order of 640mL/min.

Once the Y has been concentrated on the cartridges the eight sets of stacked cartridges are moved into a Hidex Q-ARE 100 (Hidex Oy, Finland) system that allows for an automatized multi-step purification of the Y.

The respective steps are shown in the following graph, while the subsequent figure gives an overview which impurity is eliminated in which step.



Scheme of the Y separation from seawater



Separation of Y from interferences on DGA

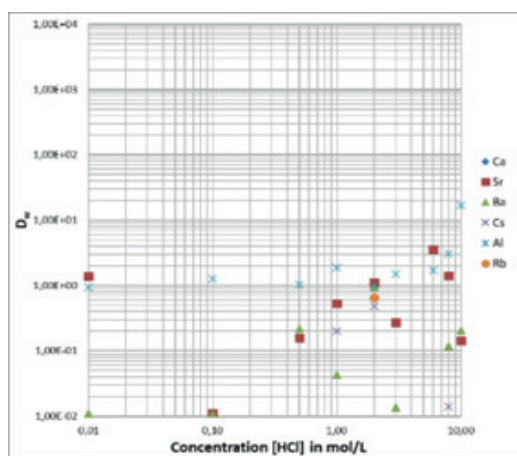
The separation on the Q-ARE 100 is performed at 5mL/min. Y is eluted from the eight channels (16 cartridges) with 40mL 0.1M HCl per channel cartridges (overall 320mL).

To lower the volume of the Y eluate, and to allow further purification, a set of two stacked 1mL DGA cartridges is moved to the SALT-100 unit. The Y eluate (320mL of 0.1M HCl) is adjusted to 3M HNO_3 and loaded onto the cartridges at 80mL/min. The stacked cartridges that are now containing the Y are then moved back to the Q-ARE 100 unit to go through an additional purification cycle (see above) and finally elution in 40mL 0.1M HCl.

The Y-90 sample is prepared for counting by precipitation at pH 10 and redissolution in 10mL 0.1M HNO_3 . An aliquot of the sample is taken and analysed by ICP-OES for Y yield determination. The remaining sample is analysed for 10 x 40minutes in Cerenkov mode.

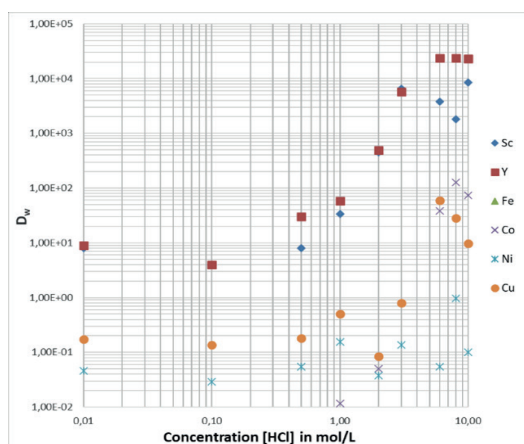
For 50L of seawater and the described counting modalities an MDA of $0.16 \pm 0.1 \text{ mBq/m}^3$ was reported by Kim et al.. Y yields were typically greater 90%.

The following six graphs show the selectivity of the TK227 Resin for a wide range of elements in HCl and the six graphs following these in HNO_3 . All D_w values shown in these graphs were obtained through ICP-MS measurements.



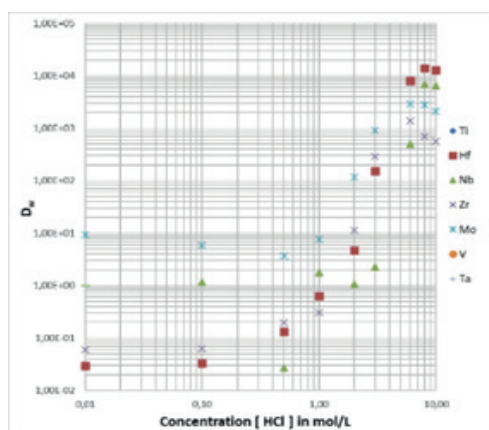
D_w values, TK227, selected elements, HCl

None of the tested elements show significant retention on TK227 in HCl.



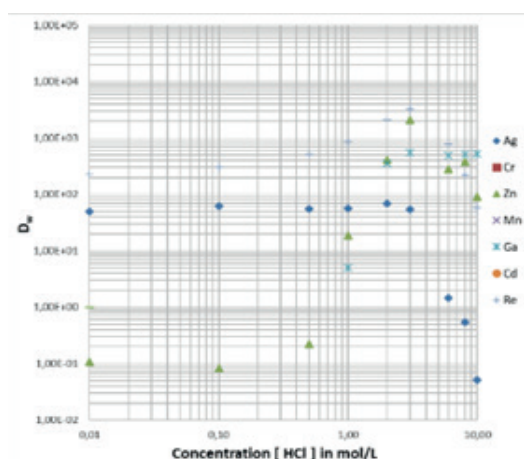
D_w values, TK227, selected elements, HCl

Y and Sc show a behaviour similar to the lanthanides, high to very high retention in high HCl, elution in dilute HCl. Tested transition metals are generally not well retained, with the exception of Cu and Co which show some retention at 6-8M HCl.



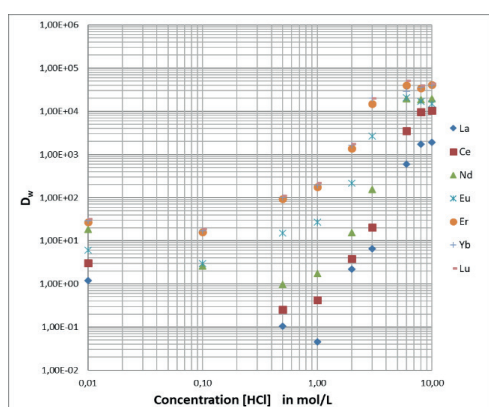
D_w values, TK227, selected elements, HCl

Higher valent elements such as Hf, Zr, Mo and Nb are well retained at elevated HCl concentrations. At lower HCl concentrations the retention decreases quite steeply.



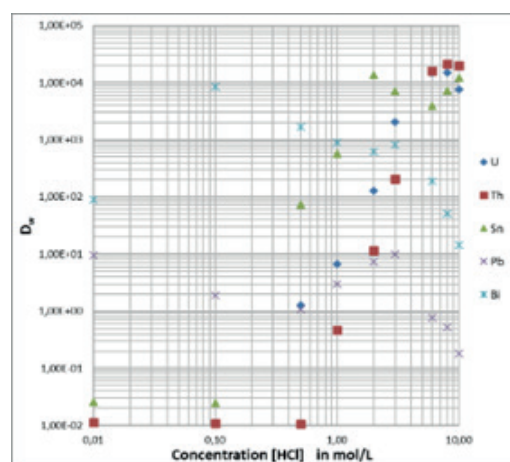
D_w values, TK227, selected elements, HCl

Elements such as Zn, Ga show fairly high retention from 2M HCl onwards. Ag is to a certain extent retained at lower HCl concentrations, while showing strong decrease at high HCl concentrations.



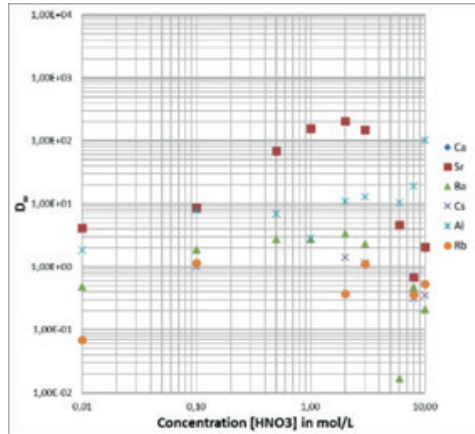
D_w values, TK227, selected elements, HCl

All lanthanides are well retained at high HCl concentrations. To be noted, the retention of the heavy lanthanides is significantly higher than the retention of light lanthanides. In general, an elution of the lanthanides may be performed at low HCl concentrations.



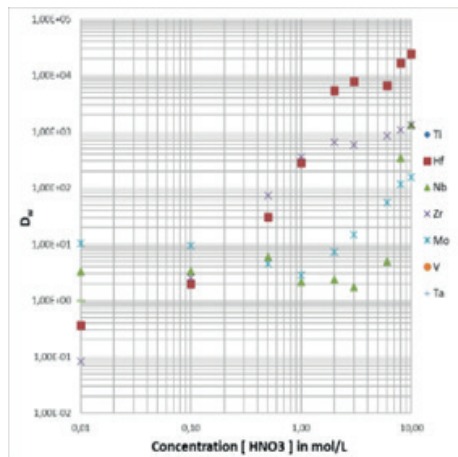
D_w values, TK227, selected elements, HCl

U, Th and Sn show negligible retention at less than 2M HCl, above this HCl concentration though retention increases strongly. Bi shows contrary behaviour, strong retention at low HCl and decreased retention at high HCl concentration (e.g. 10M).



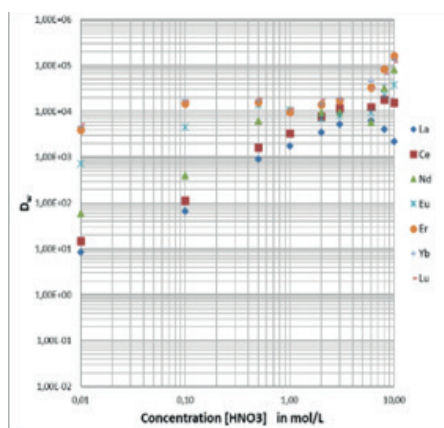
D_w values, TK227, selected elements, HNO_3

Out of the tested elements only Sr is well retained on TK227 between 1M and 3M HNO_3 .



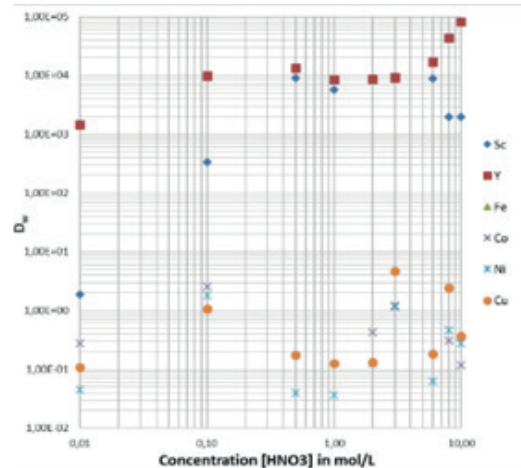
D_w values, TK227, selected elements, HNO_3

Similar to HCl, higher valent elements such Hf, Zr, Nb and to a lesser extent Mo, are well retained at elevated HNO_3 concentrations.



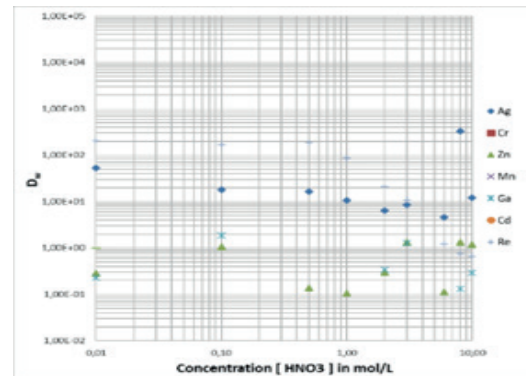
D_w values, TK227, selected elements, HNO_3

All lanthanides are very well retained at elevated HNO_3 concentrations. Heavier lanthanides remain well retained even at low acid concentrations. Overall dilute HCl seems to represent the best option for lanthanide elution.



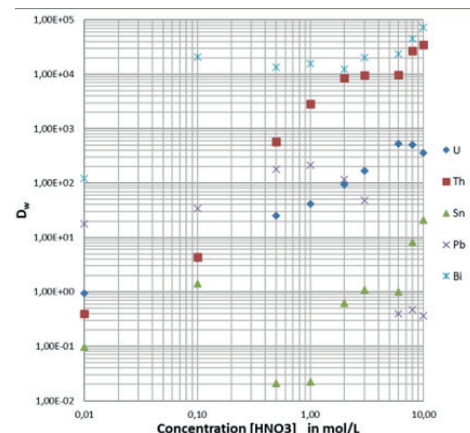
D_w values, TK227, selected elements, HNO_3

Y and Sc are very well retained at high HNO_3 concentrations. While Sc retention decreases at low HNO_3 concentrations, the Y retention remains very high. Indeed, Y requires elution with dilute HCl, similar to the lanthanides. Tested transition metals showed no significant retention in HNO_3 .



D_w values, TK227, selected elements, HNO_3

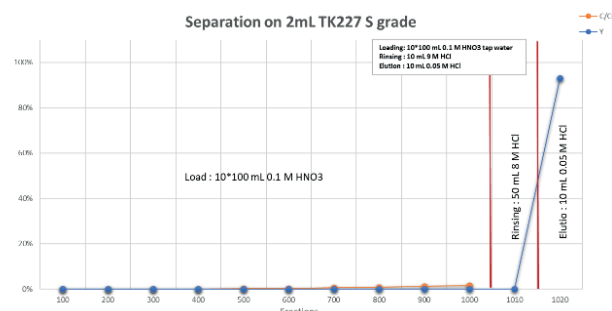
None of the tested elements show strong retention on TK227 from HNO_3 .



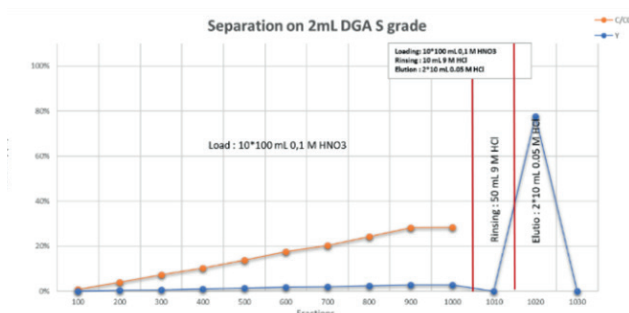
D_w values, TK227, selected elements, HNO_3

U and Th show good to high retention on the TK227 from elevated HNO_3 . Bi is overall well retained, even at rather low HNO_3 concentrations. Pb is quite well retained from 0.5 – 1M HNO_3 .

The following graphs show the results of elution studies with the aim of concentrating and separating Y from various types of water samples. In these tests 2mL cartridges and 1L water samples acidified to 0.1 – 0.2M HNO_3 . Were employed. To show the effect of the ionic liquid added to the composition of the TK227 Resin results are compared to results obtained with DGA, N Resin. Important: it should be noted that Y needs to be present in mg amounts in such experiments, at trace concentrations Y will not be eluted quantitatively.

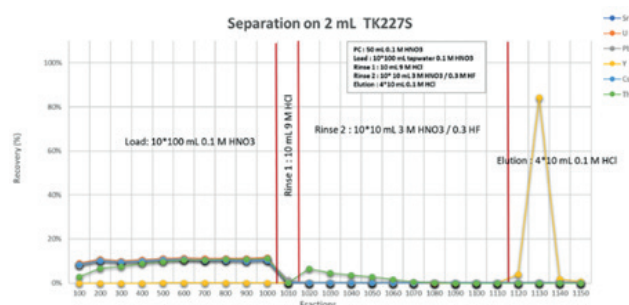


Elution study, 1L tap water, loading from 0.1M HNO_3 , Y separation, 2mL TK227 Resin cartridge



Elution study, 1L tap water, loading from 0.1M HNO_3 , Y separation, 2mL DGA, N Resin cartridge

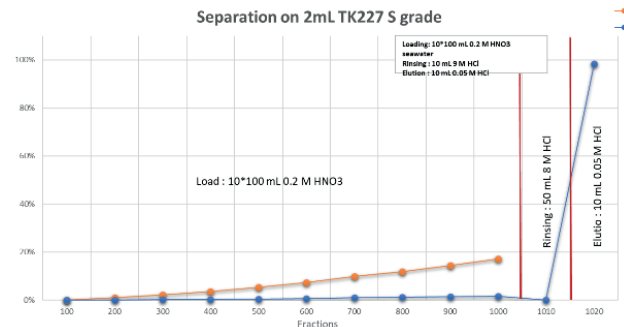
It can be seen that while TK227 Resin allows for quantitative retention of Y from 0.1M HNO_3 , DGA, N Resin shows significant breakthrough under these conditions.



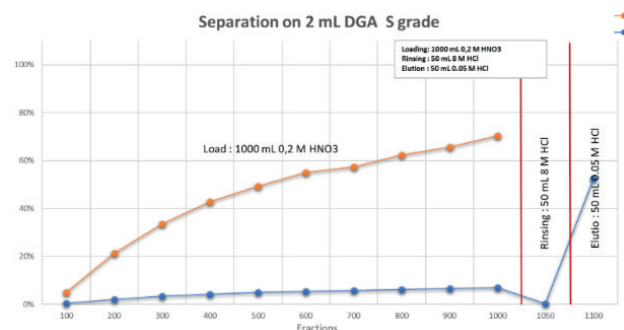
Elution study, 1L tap water, loading from 0.1M HNO_3 , Y separation from selected elements, 2mL TK227 Resin cartridge

The elution study above shows that Y can be preconcentrated from tap water (0.1M HNO_3) and then purified. The shown separation scheme allows for the removal of all considered impurities.

Sea water samples show a similar trend to tap water samples. Due to the high matrix load these samples have to be acidified a more strongly (0.2M HNO_3) to achieve elevated yields.



Elution study, 1L sea water, loading from 0.2M HNO_3 , Y separation, 2mL TK227 Resin cartridge



Elution study, 1L sea water, loading from 0.2M HNO_3 , Y separation, 2mL DGA, N Resin cartridge

For sea water samples TK227 shows some breakthrough, too – much less than DGA, N though. In order to obtain even higher Y yields for sea water samples the HNO_3 concentration of the sample before load needs to be increased further.

Main applications

- Determination of Sr-90 in large seawater samples via Y-90

Guard Resin

The Guard Resin is a hydrophobic, highly crosslinked, porous polydivinylbenzene based adsorbent. Due to its high hydrophobicity it will remove certain organic impurities, notably organic impurities that are hydrophobic, more efficiently than e.g. the Prefilter Resin. The Guard Resin is generally used in reversed phase chromatography and solid phase extraction, and for the adsorption of biomolecules of up to 14kDa. It has a surface area of $> 600\text{m}^2/\text{g}$ and a typical porosity in the order of $300 - 500\text{\AA}$. The resin shows high mechanical and chemical stability, and it may be used over the whole pH range.

The Guard Resin is TSE/BSE/GMO free.

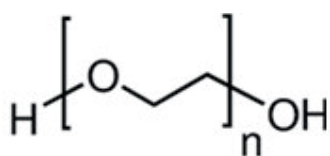
Main applications

- Removal of organic impurities



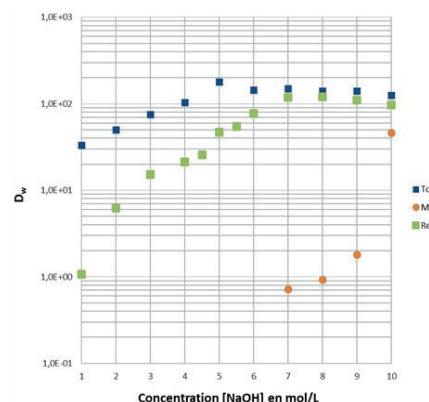
TK202 Resin

The TK202 Resin is based on Polyethyleneglycol (PEG) groups, with high molecular weight, that are covalently bound onto a polymer support.



PolyEthyleneGlycol (PEG).

The TK202 Resin is based on an aqueous biphasic system (ABS) extraction mechanism with the covalently bound PEG acting as solid separation support. In presence of aqueous solutions with high ionic strength and high content of water-structuring (kosmotropic) anions like SO_4^{2-} , CO_3^{2-} , OH^- , as well as MoO_4^{2-} or WO_4^{2-} , it will extract chaotropic ions, according to Spear et al. notably TcO_4^- and ReO_4^- , while other non-chaotropic elements will not be retained, molybdenum being an important example.

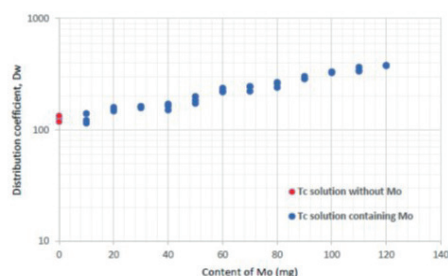


D_w values for Tc, Re and Mo on TK202 Resin, at varying NaOH concentrations. Tc data taken from Cieszykowska et al.

Accordingly, Tc (and Re) retention will improve with increasing concentration of these anions. The graph above shows D_w values for Tc, Re and Mo on TK202 Resin at increasing OH^- concentrations.

Ideally the NaOH concentration should be between 5 and 7M NaOH during load and rinse, as Tc (and Re) retention is highest while Mo retention is very low.

As mentioned before, MoO_4^{2-} itself is a kosmotropic anion. Accordingly, increasing its concentration will lead to higher Tc (and Re) retention in ABS systems like the TK202 Resin, as shown below. A distinct increase of the Tc retention with increasing amounts of Mo is observed.



D_w values for Tc in 5M NaOH using 40mg TK202 Resin, increasing amounts of Mo. Data taken from Cieszykowska et al.

This is particularly relevant in case of the separation of Tc from elevated amounts of Mo (e.g. irradiated Mo targets).

Cieszykowska et al. estimated from column studies that 6 to 8g of Mo per g of TK202 Resin allow obtaining high Tc recovery (> 90%). Further increasing the amount to 12g Mo/g of resin lead to a decrease of the Tc recovery to ~82% in their experiments.

The retained Tc and Re can then be eluted with water as the ABS systems breaks down, due to the low concentration of kosmotropic anions, under these conditions.

As mentioned, one potential application of the TK202 Resin is the separation of Tc-99m from irradiated Mo targets. Accordingly, its high selectivity for Tc over Mo, and the fact that the presence of elevated amounts of Mo in solution increases the Tc retention, makes the TK202 Resin particularly suitable for this type of applications.

Indeed, besides the production of Mo-99 for the fabrication of Mo-99/Tc-99m generators via U-235 fission there are a number of other ways to produce Mo-99, and thus Tc-99m. Three of these methods are based on the irradiation of Mo targets:

- Neutron activation of Mo-98 via (n, g) reactions ($\text{Mo-98} \xrightarrow{n, \gamma} \text{Mo-99}$), preferably performed in a reactor with high neutron flux.
- Photon-induced (γ, n) reaction of Mo-100 ($\text{Mo-100} \xrightarrow{\gamma, n} \text{Mo-99}$) using photons (γ) obtained through irradiation of heavy targets (converter) e.g. with electron beams.
- Direct Tc-99m production on a cyclotron using Mo-98 targets ($\text{Mo-98} \xrightarrow{p, 2n} \text{Tc-99m}$). This latter method represents, with respect to the short half-life of Tc-99m, certain logistical challenges and will generally rather allow for supplying users close to the production facility. This method requires a clean, and very rapid, separation of the produced Tc-99m from the target material.

The first two described methods tend to result in Mo-99 of limited specific activity, especially compared to Mo-99 obtained from U fission. Accordingly using them e.g. in alumina column based generator systems will, due to the generally limited Mo capacity of these columns, result in rather lower activity Mo-99/Tc-99m generators.

In such cases employing a resin, such as the TK202 Resin, to extract the Tc-99m originating from Mo-99 decay while letting Mo pass through ("inverted generator") is often a preferred option.

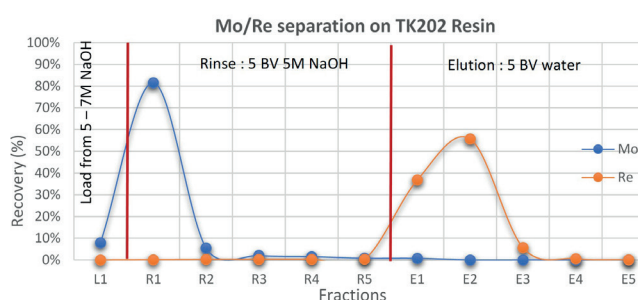
It should be noted that in all three cases the recovery and recycling of the enriched Mo is of very high importance due to the pricing and limited availability of the target material.

The production methods described above all require the use of a resin with high selectivity for Tc over large amounts of Mo. Ideally, as the Mo targets are very frequently dissolved in NaOH solutions of elevated concentration, the resin should show this selectivity under these conditions.

This is the case for the TK202 Resin. As indicated before, Tc may then be recovered using water, although further separation steps will be necessary to adjust pH and Na^+ concentration of the final product.

Initial elution studies using Re instead of Tc confirmed the high selectivity for Re (and, as could be confirmed in separate tests, also Tc) over Mo.

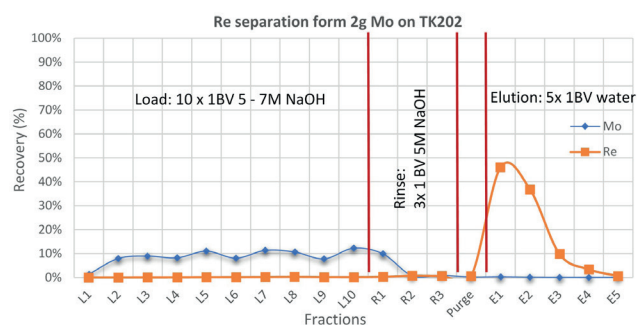
The graph below shows an elution study performed with trace amounts of Mo and Re. As it can be seen, a clean separation of both elements is obtained. Mo is removed during load and following rinses (both may be performed with 5 – 7M NaOH), while Re elutes in a small water volume.



Elution study, trace amounts of Mo and Re on a 2mL TK202 Resin cartridge, load and rinse at 1BV/min, elution at 0.25BV/min.

As the general selectivity could be confirmed further tests were performed using larger amounts of Mo.

The elution study below shows the separation of traces of Re from 2g of Mo, as e.g. typically required in case of Tc-99m production from Mo-98 irradiation in a cyclotron, as described e.g. by Bénard et al.



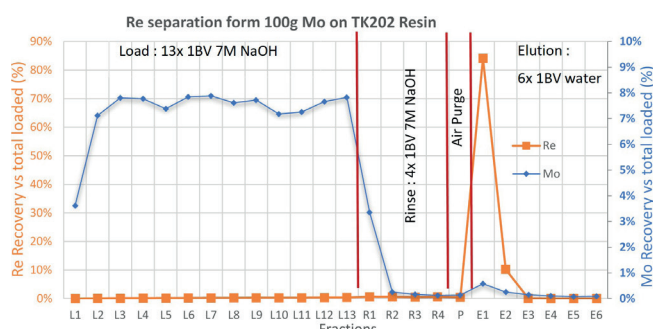
Elution study, separation of trace Re and 2g Mo on a 2mL TK202 Resin cartridge, load and rinse at 1BV/min, elution at 0.25BV/min.

It should be noted that purging the resin, e.g. with air, after the rinse and before its elution with water, to remove NaOH from the cartridge/column, is of high importance to reduce the Na^+ and OH^- load of the final Tc/Re fractions.

It could further be shown that lower flow rates during elution of the TK202 Resin will result in narrower elution peaks, and thus lower elution volumes.

As discussed previously, the TK202 Resin may also be used to extract Tc-99m from Mo-99 decay present in an alkaline solution. In such cases much larger Mo targets are generally irradiated.

With respect to this, the separation of trace Re from 100g Mo was tested. As shown below a clean separation of Mo and Re could be obtained here, too. Nevertheless, with respect to the very large amount of Mo present, an additional purification of the obtained Tc will be required.



Elution study, separation of trace amounts of Re and 100g Mo on a 75mL TK202 Resin cartridge, load at 0.5BV/min, rinse at 1BV/min, elution at 0.2BV/min.

Elution study, separation of trace amounts of Re and 100g Mo on a 75mL TK202 Resin cartridge, load at 0.5BV/min, rinse at 1BV/min, elution at 0.2BV/min.

Methods for the separation of Tc from larger Mo amounts (e.g. $\geq 200\text{g}$) are currently being tested.

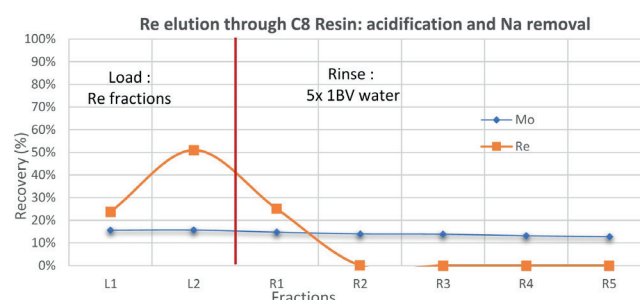
As shown e.g. by Bénard et al., one convenient option for this additional Tc purification is the use of a cation exchange resin (for Na^+ removal and pH adjustment to below pH 7), followed by an alumina cartridge (for Re/Tc concentration and further Mo removal).

Especially for the cation exchange cartridge it is important to adjust the size of the cartridge to the amount of Mo previously present in the sample/and so the size of the TK202 Resin cartridge employed.

For the size of the alumina cartridge on the other hand the amount of residual Mo will be a decisive parameter.

The graph below shows the continuation of the 2g Mo separation test shown before.

The obtained Re fractions (E1 – E4) were combined and loaded through a C8 cation exchange resin cartridge which was then rinsed with water. The load fractions and the first rinse were collected, analyzed, and then combined for the final step of the separation, as they contain all the Re (or Tc).



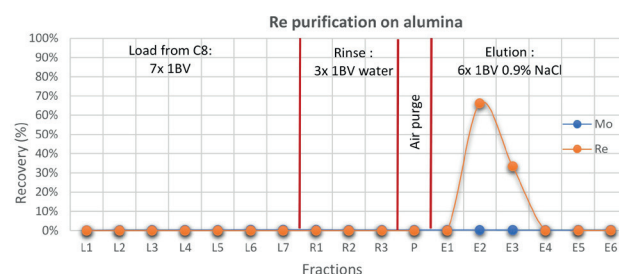
Continuation of the separation of Re from 2g Mo (see above). Na^+ removal and pH adjustment of Re fractions on 2mL C8 Resin cartridge, load and rinse at 2BV/min.

As stated before, at this stage the Re fraction should be below pH 7 (typically 3 – 5) and largely free of Na^+ cations.

It could be shown that under these conditions, acidic alumina will retain Re/Tc (and Mo). A 0.9% NaCl solution then allows eluting Re/Tc in a small volume (2 – 3 BV) while Mo remains very strongly retained, thus further improving the purity of the recovered Re/Tc.

The indicated air purge is not necessary in case of the AlOxAl Resin (acidic alumina).

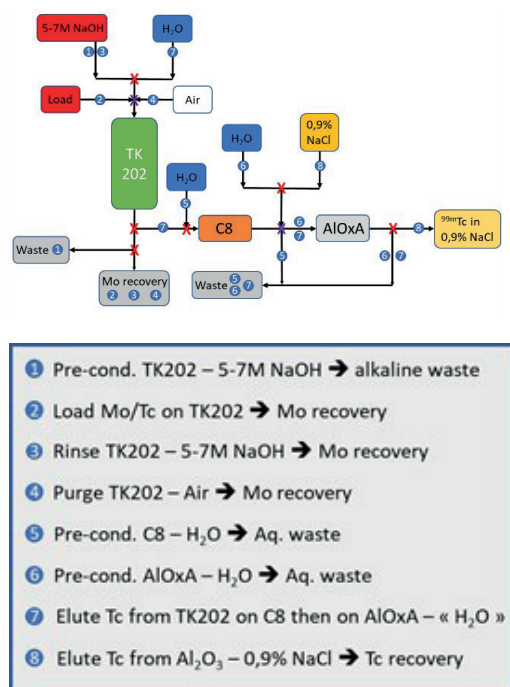
This further has the advantage of allowing to obtain the Tc in the same matrix (0.9% NaCl) as delivered by a Mo-99/Tc-99m generator.



Re concentration, purification and conversion to 0.9% NaCl solution on 1mL AlOxAl Resin at 2BV/min.

Overall, in the cold tests Re recoveries in the order of >90% could be obtained.

A schematic overview of the suggested separation method is given below. The method may be applied for the separation of Tc from Mo targets of various sizes, cartridge/column volumes will need to be adjusted accordingly.

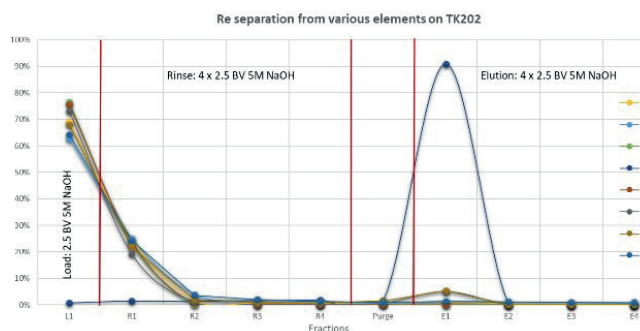


Schematic overview, Tc separation from Mo.

Besides the described radiopharmaceutical use the TK202 Resin may also be employed in radioanalytical applications, notably in the analysis of Tc-99 in samples that were solubilized via alkaline fusion.

In such cases the solubilized samples (e.g. concrete samples resulting from decommissioning work) may, after removal of insoluble material, be adjusted to 5 – 7M NaOH and then passed through TK202 Resin to separate Tc.

In order to further increase the purity of the obtained Tc fraction it might be passed, as described above, through a C8 Resin, and potentially even AlOx A Resin.



Re separation from selected elements on 2mL TK202 Resin cartridge, load and rinse at 1BV/min, elution at 0.25BV/min.

Main applications

- Separation of technetium
- Separation of rhenium

Further to the TK202 Resin TrisKem also supplies C8 Resin and AlOx A Resin.

All resins are available in various columns and/or cartridges of different sizes (depending on the size of the Mo target). Please contact us for further information.



TK-TcScint Resin

The TK-TcScint Resin is the first of a range of resins based on "Impregnated Plastic Scintillation microspheres" developed by García, Tarancón and Bagán at the University of Barcelona.

This range of new products will be comprised of plastic scintillation microsphere (PSm), supplied by the group at the University of Barcelona, that are impregnated with selective extractants.

The TK-TcScint Resin is, as the name indicates, mainly dedicated to the quantification of Tc-99. The extractant used in its fabrication is Aliquat336, it further contains small amounts of a long-chained alcohol. Accordingly, its selectivity will generally be very similar to the TEVA Resin.

The figures below (a and b) show SEM pictures of the non-impregnated PSm, and the impregnated microspheres (TK-TcScint), respectively.

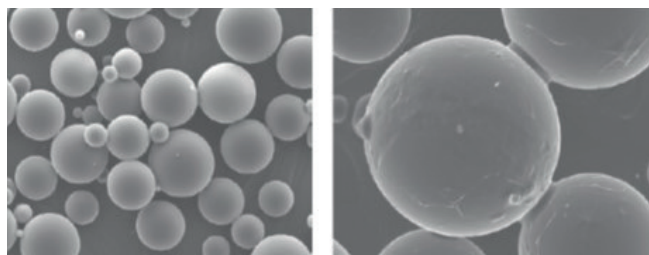


Figure1 (a and b): left - non-impregnated plastic scintillation microspheres (PSm), right - impregnated PSm (TK-TcScint Resin).
Taken from Garcia et al.

The TK-TcScint Resin is generally employed as pre-packed 2mL cartridges for use with vacuum box systems, or automatized separation equipments based on pump systems

The PSm support employed in the TK-TcScint Resin is itself acting as scintillating medium, this allows for a direct measurement of the Tc-99 retained on the cartridge, no elution and mixing of the eluate with a liquid scintillation cocktail is necessary.

This has a number of advantages:

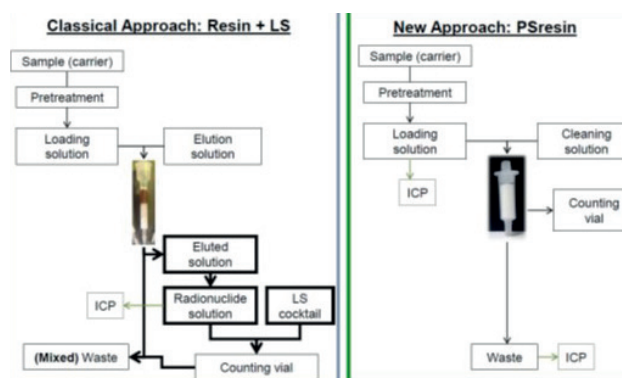
- Gain of time which is particularly interesting in emergency situations
- No mixed liquid radioactive waste
- No Tc elution with HNO_3 of elevated concentration and no evaporation / aliquoting of the eluate
- No cutting of columns or cartridges to push the resin into LSC vials

Especially the latter two points are interesting in terms of radiation protection when samples of elevated activity are being analyzed.

Ideally the chemical yield is determined via ICP-MS or ICP-OES using Re as internal standard.

The following graph compares this new approach based on impregnated PSm Resins such as the TK-TcScint Resin with classical methods.

In order to easily handle and avoid contaminating the LSC counter the cartridges should be placed in a standard 20mL LSC vial for its measurement.



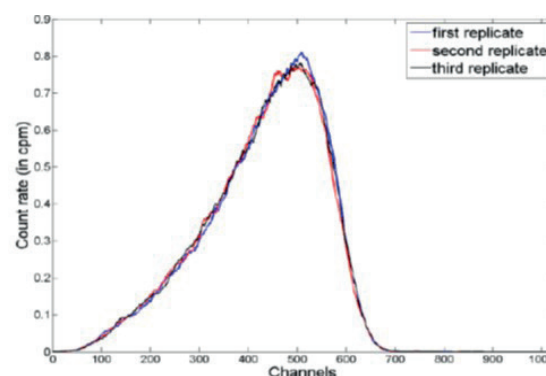
Overview classical radioanalytical method and PS Resin approach. Taken from Garcia et al.

Typical samples analysed include urine and various types of water sample. In case of surface water samples generally a breakthrough volume of > 200mL can be achieved using 2mL cartridges, making this technique not only interesting in emergency situations or as screening tool in decommissioning, but also for use in routine biomedical or environmental monitoring. For water samples the chemical yield is generally >98.8%.

The detection efficiency for Tc-99 obtained with the TK-TcScint Resin is very high, in the order of 89.5(0.6)%, while the background of the standard 2mL cartridges is low with ~1.09CPM (obtained in a QuantulusTM detector in the high-energy and low-coincident bias configuration).

Further the TK-TcScint Resin cartridges show reproducibly low quench with a mean SQP(E) of 787(7).

The figure below shows typically obtained Tc-99 spectra, as can be seen spectra obtained for three replicates match very well.



Liquid Scintillation Spectrum of Tc-99 on TK-TcScint.
Taken from Garcia et al.

The analysis of water samples such as e.g. river and sea water (typically 50mL) using TK-TcScint Resin cartridges is rather straightforward.

After filtration, if necessary, samples are heated to 90°C for 60min after addition of a few mL of 30% H₂O₂ to assure that Tc is present as pertechnetate. The solutions are then adjusted to 0.1M HCl using conc. HCl. Once the samples reach room temperature, they are ready for separation.

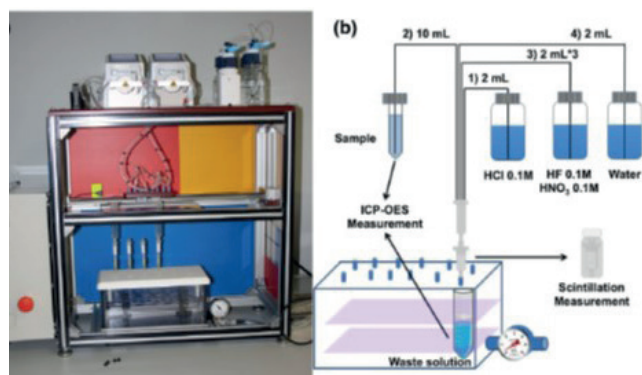
After loading of the sample the cartridge is typically rinsed successively with 0.1M HCl, 0.1M HNO₃/0.1M HF (only necessary in case Th is expected to be present) and finally water.

These rinses allow eliminating possible interferences while Tc (and the internal standard Re) remain on the cartridge. Load and rinse fractions are combined and analysed for Re content to allow calculating the chemical yield of the separation. The TK-TcScint Resin cartridge can then be directly counted on an LSC counter.

The authors found very good match between expected and measured activities, for the spiked water samples as well as for two spiked MAPEP samples.

Using 50mL samples and 180min counting time allowed the authors obtaining a limit of detection of 0.15Bq/L.

As could further be shown by the authors, this method can easily be automatized. In their case they developed their own separation unit called OPENVIEW-AMSS, a modular, vacuum box based equipment. They could show that both, manual and automatized separations allow for obtaining high chemical yields and detection efficiencies, no significant differences were observed when analysing samples in parallel. However, with respect to hands-on time and radiation protection automatization provides significant advantages.



OPENVIEW-AMSS system and scheme of a typical separation of water samples. Taken from Coma et al.

Further to the unit developed by the authors the TK-TcScint Resin cartridges are also compatible with commercially available equipment such as the Hidex Q-ARE 100.

Besides water samples urine samples were also analysed using TK-TcScint Resin cartridges.

With respect to their higher matrix load this kind of samples requires a thorough sample pre-treatment. The described method is based on 100mL urine samples that are first wet-ashed using conc. HNO₃, followed by an additional ashing step in a muffle furnace at 550°C.

The obtained ash is then dissolved in 3mL of conc. HNO₃ and diluted to 100mL using deionized water. To assure Tc is present as pertechnetate the solutions are heated, after addition of a few mL hydrogen peroxide, to 90°C for 60min. As described before, Re was used as internal standard.

By analysing spiked urine samples the authors could show that accurate results can be obtained using the impregnated PSm approach. A minimum detectable activity (MDA) of 0.036Bq/L for 100mL samples and 24h counting was reported.

Further to the analysis of Tc-99, Bagán et al. showed that Aliquat 336 impregnated PSm resins may also be used for the analysis of [¹⁴C]SCN⁻ used as radiotracer for study of oil reservoir dynamics.

With respect to the selectivity of the Aliquat extractant, the compound giving TEVA Resin its selectivity, a use of the TK-TcScint Resin cartridges for the screening of other radioelements such as e.g. Pu isotopes or Po-210 seems well possible.

Main applications

- Separation and LSC measurement of technetium



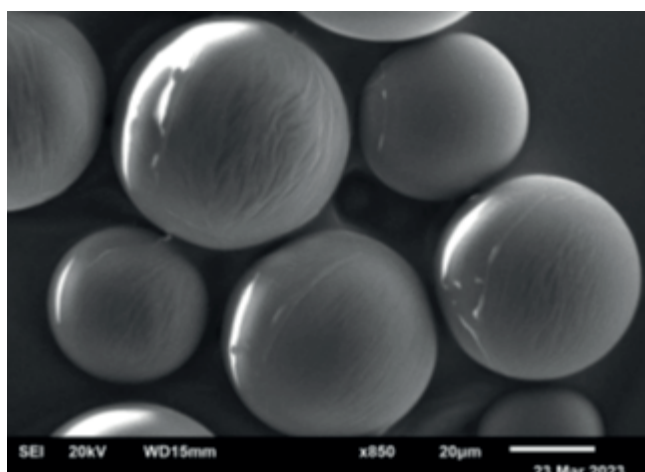
TK-SrScint Resin

The TK-SrScint Resin is, like the TK-TcScint, a resin based on “Impregnated Plastic Scintillation microspheres” developed by Tarancón, and Bagán at the University of Barcelona.

These new materials consist of plastic scintillation microspheres (PSm), supplied by the group at the University of Barcelona, which are impregnated with selective extractants.

The TK-SrScint Resin is designed for use in similar separation methods to those employing SR or TK102 Resins. The selective extractant used in the production of this PS resin is a crown ether (also used in SR Resin) dissolved in a fluorinated alcohol (used in TK102 Resin). Consequently, its selectivity will generally be very similar to that of the aforementioned resins, which are primarily used for Sr or Pb separations.

The figure below shows SEM pictures of the impregnated microspheres (TK-SrScint).



Impregnated PSm (TK-SrScint). Courtesy of Tarancon et al.

The TK-SrScint is available as pre-packed 2mL cartridges for use with vacuum box systems or systems that can be easily automated using pump systems.

As with the also available TK-TcScint Resin, the PSm support acts as a scintillating medium, allowing for direct measurement of the isotope retained on the cartridge.

Thus, there is no need to elute the target radionuclide and mix the eluate (i.e. radioactive solution) with a liquid scintillation cocktail (i.e. organic product). Incorporating this PS resin thus helps to reduce the amount of mixed (radioactive + organic) waste generated during the determination procedure.

This has a number of advantages:

- Less hands-on time consumed which is particularly interesting in emergency situations
- No mixed liquid radioactive waste
- No Sr, Pb or Ba elution required and no evaporation / sample preparation of the eluate
- No cutting of columns or cartridges to push the resin into LSC vials

Especially the latter two points are interesting in terms of radiation protection when samples of elevated activity are being analyzed.

Ideally the chemical yield is determined via ICP-MS or ICP-OES using Sr as internal standard (comparison between the initial amount added/present and the strontium not retained on the PS Resin).

Like for the TK-TcScint it is advisable, to facilitate handling and avoid contaminating the LSC counter, that the cartridges are placed in a standard 20mL LSC vial for its measurement.

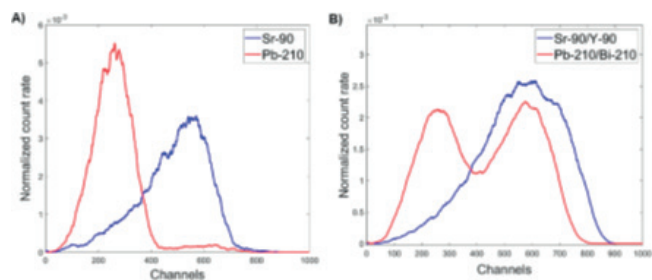
The TK-SrScint Resin has been tested in various water samples, including river, MAPEP, and CSN interlaboratory samples. Results were compared with the two most common methods for Sr-90 determination: extraction chromatography using SR Resin combined with LSC, and successive precipitations combined with LSC. Sr-90 activity evaluated using the different methods ranged from 0.49-4.9Bq/L in spiked river water samples, to 5.65-10.48Bq/L in MAPEP interlaboratory samples, and 4.1Bq/L in CSN interlaboratory samples. Moreover, in CSN samples, interferences such as Co-57, Co-60, Cs-134, Pu-238, Ra-226, Pb-210, and Ra-228 were evaluated.

Prior to applying the PS method using TK-SrScint Resin, various precipitation methods to remove Pb from the sample were investigated. Iodate precipitation was selected as the most suitable method for Pb removal, and further optimization was performed to increase Sr recovery while decreasing Pb presence (~3% remaining). The final procedure included precipitating Pb by adding iodate and calcium as coprecipitating agents and boiling of the sample. Once Pb was removed, Sr was precipitated using hydrogen phosphate in a basic medium, and the resulting precipitate was dissolved with 8M HNO₃ for loading the sample onto the TK-SrScint Resin.

After analyzing water samples spiked with CSN, Sr chemical yields of approximately 87% were achieved. The analysis considered a background signal of 0.18CPM, and no other radionuclides, including Pb-210, were detected in the samples.

For larger samples, such as 1L spiked river water samples, a carbonate precipitation step was added to reduce sample volume. In these cases reported chemical recoveries ranged between 63% and 81%.

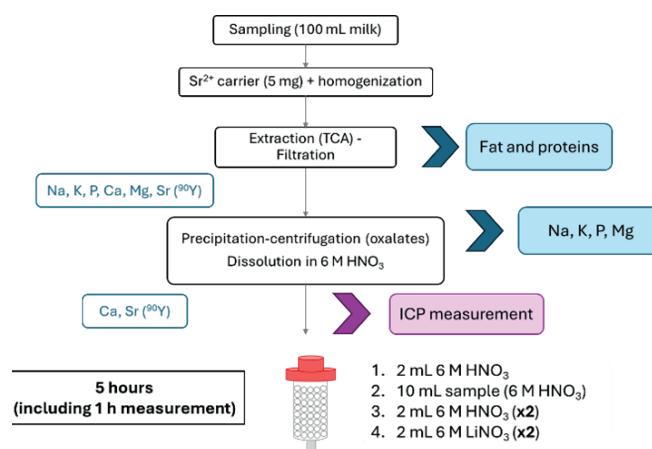
LS spectra of the target radionuclide (Sr-90) and the potential interference (Pb-210) are shown below at different times to account for Y-90 ingrowth.



Liquid Scintillation normalized Spectra of Sr-90/Y-90 (blue) and Pb-210/Bi-210 (red) on TK-SrScint at time 0 (A) and with ingrowth daughters at time >21 days (B). Courtesy of Tarancon et al.

The detection efficiency of Sr-90, evaluated at time 0, was found to range between 86% and 51% for complete and optimal window settings, respectively. The optimal window was selected to minimize the possible contribution of Pb-210 to the LS spectrum. A lower background signal of 0.3CPM, compared to standard methods, was achieved with TK-SrScint resin, resulting in a lower detection limit of 27mBq/L (1h counting time). As mentioned earlier, the inclusion of PS Resin in the Sr-90 determination method reduced the procedural turnaround time to 5-6hours.

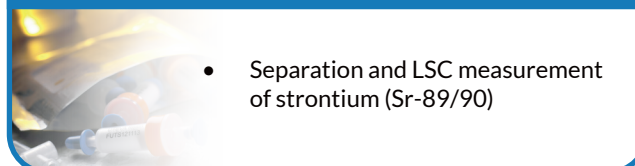
Determination of Sr-90 in milk samples has also been investigated by Tarancón et al. Their tests resulted in the detection of approximately 65% of the total Sr-90.



Sr-90 determination procedure in milk samples. Courtesy of Tarancon et al.

According to the results obtained by Tarancón et al. (2024) when testing IAEA milk powder samples, the Sr-90 activity quantified using the TK-SrScint Resin was in agreement with the reference activity. Moreover, interferences such as Ca, Na, and K were shown to be removed by observing a clear Sr-90/Y-90 spectrum and through measurements taken with ICP.

Main applications



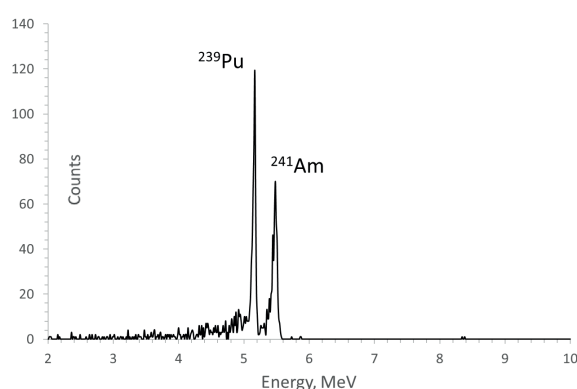
- Separation and LSC measurement of strontium (Sr-89/90)



Extractive Discs

- Range of selective impregnated membrane filters
- Ø = 25mm and 47mm Discs will be available
- Various extractant systems (TK100, TK201, GrossActinide, CU,...) in beta-testing
- More Discs under development
- For use in filtration or passive sampling set-ups

Use of TK-GrossActinide Discs for direct alpha spectrometry of aqueous samples



Alpha spectrum, 25mm TK-GrossActinide Disc,
20 mL HNO₃ at pH2 spiked with
30 mBq each Am-241 and Pu-239

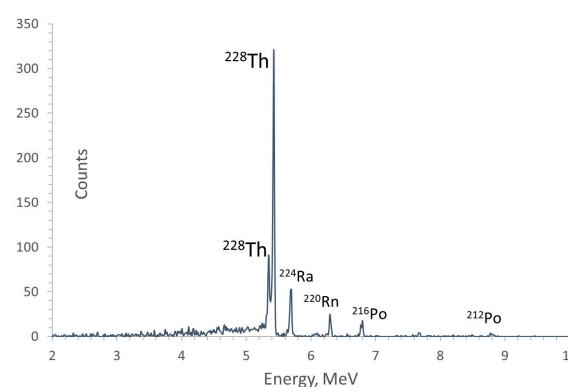
Disc preparation

Sample filtration
(at pH 1 – 2)

Drying of the disc

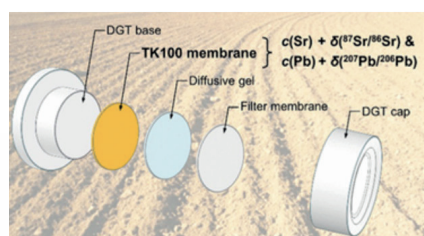
Glueing on a
rigid support

Alpha spectrometry
measurement



Alpha Spectrum,
47mm TK-GrossActinide Disc,
100 mL Pedras mineral water at pH 1

TK100 Discs for passive sampling in soils



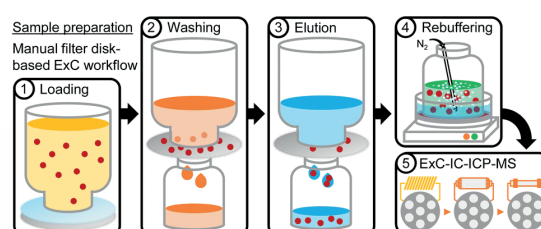
DGT set-up using TK100 Discs for Sr and Pb capture

« Selective Diffusive Gradients in Thin Films (DGT)
for the Simultaneous Assessment of Labile Sr and Pb
Concentrations and Isotope Ratios in Soils »

Anal. Chem. 2022, 94, 6338–6346

Stefan Wagner, Jakob Santner, Johanna Irrgeher, Markus Puschenreiter,
Steffen Happel, and Thomas Prohaska

TK201 Discs for Tc-99 preconcentration from waste water



Experimental setup, Tc-99 concentration and purification from
waste water using TK201 discs for preconcentration

« Quantification of technetium-99 in wastewater by
means of automated on-line extraction chromatogra-
phy – anion-exchange chromatography – inductively
coupled plasma-mass spectrometry »

J. Anal. At. Spectrom., 2024, 39, 2774-2782

Maximilian Horstmann, C. Derrick Quarles Jr, Steffen Happel,
Michael Sperling, Andreas Faust, David Clases* and Uwe Karst

TRISKEM INTERNATIONAL

ZAC de l'Eperon – 3, rue des Champs Géons – 35170 Bruz – FRANCE

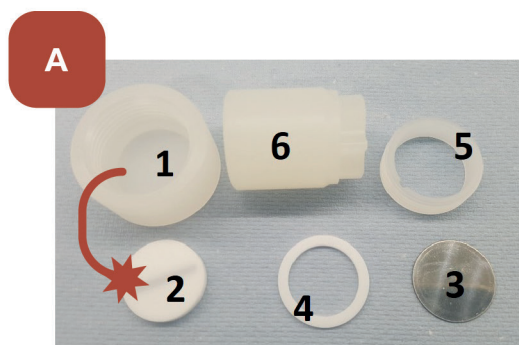
Tel +33 (0)2.99.05.00.09 – E-Mail : contact@triskem.fr

www.triskem.com

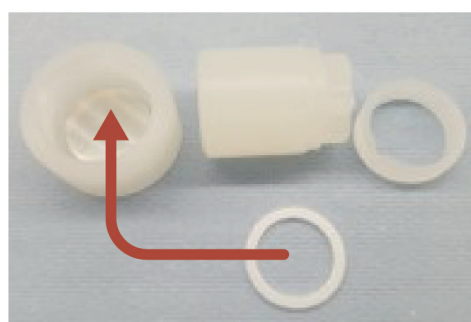
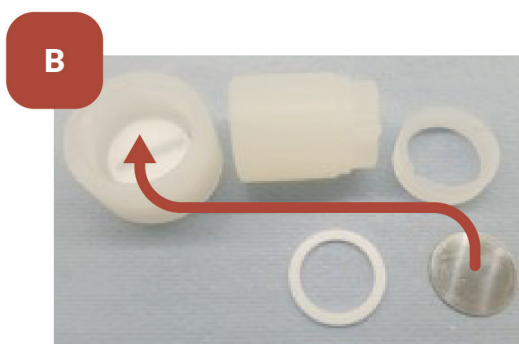


Auto-deposition KIT AC 05 ADK

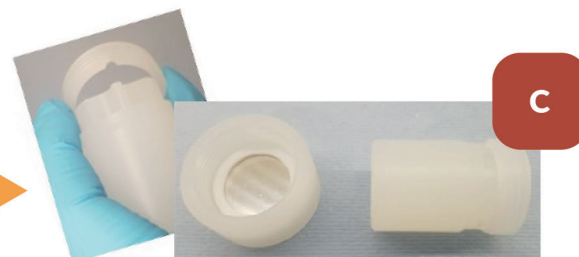
Instruction for Use



(1) Plastic rotative unit, (2) Flat circular magnet Ring, (3) Disk, (4) Ring, (5) Clamping ring, (6) Retaining clip



Insert (2) Flat circular magnet Ring into (1) Plastic rotative unit then the (3) Disk and the (4) Ring



Place the (6) Retaining clip in the (5) Clamping ring



The system is ready for deposition in a beaker on stirring plate



Tighten the (5) Clamping ring in the (1) Plastic rotative unit

REFERENCES

- J. L. Cortina and A. Warshawsky Ion Exchange and Solvent Extraction; Marinsky, J.A., Marcus, Y., Eds.; Marcel Dekker: , 1997; Vol. 13, p.195.
- M. L. Dietz, E. P. Horwitz, and A. H. Bond,» Metal in Separation and Preconcentration: Progress and Opportunities, Bond, A. H., Dietz, M. L., R.D., Eds.; ACS Symposium Series 716, ACS, , 1999, pp. 234-250.
- E. P. Horwitz, R. Chiarizia, and M. L. Dietz, Solvent Extr. Ion Extr. 1992, Vol. 10, pp. 313-336.
- E. P. Horwitz, M. L. Dietz, R. Chiarizia, H. Diamond, S. L. Maxwell, III, and M. R. Nelson, Anal. Chim. Acta 1995, Vol. 310, pp. 63-78.
- E. P. Horwitz, R. Chiarizia, and M. L. Dietz, Reactive and Functional Polym. 1997, Vol. 33, pp. 25-36.
- E. P. Horwitz and C. A. A. Bloomquist, J. Inorg. Nucl. Chem. 1972, Vol. 34, pp. 3851-3871.
- E. P. Horwitz and C. A. A. Bloomquist, J. Inorg. Nucl. Chem. 1973, Vol. 35, pp. 271-284.
- E. P. Horwitz, M. L. Dietz, R. Chiarizia, and R. C. Gatrone and A. M. Essling, R. W. Bane, and D. Graczyk, Anal. Chim. Acta 1992, Vol. 266, pp. 25-37.
- E. P. Horwitz, R. Chiarizia, M. L. Dietz, H. Diamond, and D. M. Nelson, Anal. Chim. Acta 1993, Vol. 281, 361-372.
- E. P. Horwitz and C. A. A. Bloomquist, J. Inorg. Nucl. Chem. 1975, Vol. 37, pp. 425-434.
- E. P. Horwitz, M. L. Dietz, S. Rhoads, C. Felinto, N. H. Gale and J. Houghton, Anal. Chim. Acta 1994, Vol. 292, pp. 263-273.
- W. C. Burnett, D. R. Corbett, M. Schultz, E. P. Horwitz, R. Chiarizia, M. Dietz, A. Thakkar, and M. Fern, J. Radioanal. Nucl. Chem. 1997, Vol. 226, p. 121.
- Horwitz E.P., McAlister D.R., Bond A.H., Barans R.E., Solvent Extrac. Ion Exch., 23, 219 (2005).
- Horwitz E.P., Bond A.H., Barans R.E., McAlister D.R., 27th Actinide separations Conferences, (2003).
- Moon D.S., Burnett W.C., Nour S., Horwitz P., Bond A., Applied Rad. Isot., 59, 255 (2003).
- Maxwell, S.L., presented at Eichrom's North American Users' Meeting, , May 3, 2005, see www.eichrom.com.
- Cahill D. F., Peedin L. M., presented at 41st Annual Conference on Bioassay, Analytical and Environmental Chemistry, Eichrom workshop, Boston, MA - USA
- Esser B.K. et al., Anal. Chem., Vol.66, 1736 (1994)
- McAlister D., Horwitz E.P., Solvent Extrac. Ion Exch, 25 (6), 757-769 (2007)
- A. Zulauf, S. Happel, M. B. Mokili et al, J. Radanal Nucl Chem, 286(2), 539-546
- P E Warwick, A Zulauf, S Happel, I W Croudace, presented at the 11th ERA Symposium, 16/09/2010, Chester (UK), see www.triskem.com
- C. Dirks, B. Scholten, S. Happel et al., J Radioanal. Nucl. Chem, 286 (2010) 671-674
- C. Dirks, S. Happel, presented at the Triskem International users group meeting, 14/09/2010, Chester (UK), see www.triskem.com
- Chiarizia, R., Sep. Sci. Technol., 32, 1997, 1 - 35
- Kim G., Burnett W.C., Horwitz E.P., Anal. Chem., 72, 2000, 4882-4887
- Shaw D.R. et al., JOM, July 2004, 38 - 42
- Hines, J.J.; et al, Sep Sci Technol, 30(7-9), 1995, 1373 - 1384
- Surbeck H., presented at ICRM Conference on Low Level Radioactivity Measurement Techniques 18-22nd October 1999, Mol, Belgium, see www.nucfilm.com
- Mendes M, Aupiais J, Jutier C, Pointurier F. Anal Chim Acta. 2013 May 30;780:110-6
- Horwitz E.P., Dietz M.L., Rhoads S., Felinto C., Gale N.H., Houghton J.; Analytica Chimica Acta, Vol.292, p263-273 (1994)
- Nottoli E, Biennu P, Labet A, Bourlès D, Arnold M, Bertaux M. Appl Radiat Isot. 2014 Apr;86:90-6. doi: 10.1016/j.apradiso.2014.01.010.
- Horwitz P, McAlister D. Old Separations on New Resins and New Separations on Old Resins. RRMCM, Destin, FL 2008
- Sebesta F, Stefula V (1990) Composite ion exchanger with ammonium molybdophosphate and its properties. J Radioanal Nucl Chem 140(1):15 - 21
- Brewer et al. (1999). Czechoslov J Phys 49(S1):959-964
- Kamenik et al. (2012) J Radioanal Nucl Chem. DOI 10.1007/s10967-012-2007-4
- Kamenik J et al. (2009) Long term monitoring of Cs-137 in foodstuffs in the Czech Republic. Appl Radiat Isot 67(5):974-977
- Bartuskova et al. (2007) Ingestion doses for a group with higher intake of Cs-137. IRPA regional congress for Central and Eastern Europe, Brasov, Romania
- E. P. Horwitz, M. L. Dietz, R. Chiarizia, H. Diamond, S. L. Maxwell, III, and M. R. Nelson, Anal. Chim. Acta 1995, Vol. 310, pp. 63-78.
- Mark L. Dietz, Julie A. Dzielawa, Ivan Laszak, Blake A. Young and Mark P. Jensen: „Influence of solvent structural variations on the mechanism of facilitated ion transfer into room-temperature ionic liquids”, Green Chemistry, 2003, 5, 682-685
- Dirks et al.: “On the development and characterisation of an hydroxamate based extraction chromatographic resin”. Presented at the 61st RRMCM, October 25th - 30th, 2015, Iowa City, IA, USA
- A. Bombard et al. “Technetium-99/99m New Resins Developments For Separation And Isolation From Various Matrices”, presented at the ARCEBS 2018, 11-17/11/18 - Ffort Raichak (India)
- S. Happel: “An overview over some new extraction chromatographic resins and their application in radiopharmacy” presented on the 4th of June 2019 at the 102nd Canadian Chemistry Conference and Exhibition (CCCE 2019) in Quebec City, QC
- S.K. Spear et al., Ind. Eng. Chem. Res., 2000, 39, 3173 - 3180, <https://doi.org/10.1021/ie990583p>
- I.Cieszykowska et al.: “Separation of 99mTc from low specific activity 99Mo”, poster ID 195 presented at the ISTR 2019, October 28 - November 1, Vienna, Austria
- IAEA Nuclear Energy Series, No. NF-T-5.4.; “Non-HEU Production Technologies for Molybdenum-99 and Technetium-99m”, INTERNATIONAL ATOMIC ENERGY AGENCY, Vienna, 2013
- F. Bénard et al, Journal of Nuclear Medicine, 2014, <https://doi.org/10.2967/jnumed.114.143834>
- C. Dirks et al., Presented at the 61st RRMCM, October 25th - 30th, 2015, Iowa City, IA, USA

- S. Happel: "An overview over some new extraction chromatographic resins and their application in radiopharmacy", Version 12/06/2021, https://www.triskem-international.com/scripts/files/60c4a63e7a4475.44279512/tki_rp_210612.pdf, accessed on 15/07/2021
- J. P. Holland et al., Nucl Med Biol., 36(7), 2009, 729–739; doi:10.1016/j.nucmedbio.2009.05.007
- Nair M, et al., Eur J Nucl Med Mol Imaging. 2017;44(Suppl 2):S119–956.
- Stefano Riga et al., Physica Medica, Volume 55, 2018, 116–126. <https://doi.org/10.1016/j.ejmp.2018.10.018>
- Rodnick, M.E. et al., EJNMMI radiopharm. chem. 5, 25 (2020). <https://doi.org/10.1186/s41181-020-00106-9>
- Thisgaard, H. et al., EJNMMI radiopharm. chem. 6, 1 (2021). <https://doi.org/10.1186/s41181-020-00114-9>
- V. Radchenko et al., Journal of Chromatography A, Volume 1477, 2016, 39–46, <https://doi.org/10.1016/j.chroma.2016.11.047>.
- Mario Malinconico et al., Journal of Nuclear Medicine May 2018, 59 (supplement 1) 664. https://jnm.snmjournals.org/content/59/supplement_1/664
- Coma et al., Journal of Radioanalytical and Nuclear Chemistry (2019) 321:1057–1065. <https://doi.org/10.1007/s10967-019-06659-7>
- Barrera et al., Analytica Chimica Acta 936 (2016) 259–266. <https://doi.org/10.1016/j.aca.2016.07.008>
- Tarancon et al. "A new plastic scintillation resin for single-step separation, concentration and measurement of ⁹⁹Tc", presented at the NRC9 (29/08/16 – 2/09/16, Helsinki, Finland)
- Hidex eBook "Liquid Scintillation Measuring Procedures: New Developments" <https://hidex.com/ebooks/liquid-scintillation-measuring-procedures/measuring-procedures/radionuclides-from-nuclear-fission-activities/2-3-14-tc-by-rad-disk-and-psresins/>
- J. Garcia & A. Tarancon, "Radionuclide determinations with PS Resin MASS WaterRadd", presented at the European Users Group Meeting in Cambridge (UK) - 21/09/2018, https://www.triskem-international.com/scripts/files/5bae2550c30ed4.50583030/11_j-garcia_a-tarancon_radionuclide-determinations-with-ps-resin_mass_waterradd.pdf
- H. Bagán et al., Analytica Chimica Acta, 736, 2012, 30–35, <https://doi.org/10.1016/j.aca.2012.05.045>
- Illarion Dovhy, Marine Bas, Nora Vajda et al.: "Characterization of new crown-ether containing TK102 Resin for the separation of Sr, Pb and Ba/Ra", Poster presented at the 14th International Symposium on Nuclear and Environmental Radiochemical Analysis from 12 – 15/09/2022 in York (UK). <https://www.triskem-international.com/scripts/files/63317f16990d61.93025432/poster-tk102---v1.pdf>
- J. Svedjehed et al.: "New extractant impregnated iTLC-SG paper facilitates improved TLC analysis for Cu radiolabelled peptides", poster presented at TERACHEM 2022, 14 – 17 September 2022, Bressanone (Italy). Poster available online on our website.
- J. Svedjehed, K. Gagnon. A quest for simplicity: Automated cassette-based purification of [⁶¹Cu]CuCl₂ from solid Ni targets using a single time-list. Nucl Med Biol, 108–109, S1 (2022), P-220, ppS170.
- C. Bailly et al. Selective impregnated Membrane Filters for Direct Alpha Spectrometric Analysis of Actinides; Applied Radiation and Isotopes, in preparation
- M. Snow, T. Holschuh, L. Squires et al.: Rapid Analytical System For Quantifying Actinides and Fission Products From Highly Radioactive Chloride Molten Salts, <https://doi.org/10.21203/rs.3.rs-9558897/v1>
- S. Wagner, J. Santner, J. Irrgeher et al. Selective Diffusive Gradients in Thin Films (DGT) for the Simultaneous Assessment of Labile Sr and Pb Concentrations and Isotope Ratios in Soils. Anal Chem, 2022, 94, 16, 6338 – 6346 <https://doi.org/10.1021/acs.analchem.2c00546>
- Wagner, S., Santner, J., Puschenreiter, M. et al. Prediction of radiogenic Sr and Pb isotope signatures in plants using diffusive gradients in thin films. Anal Bioanal Chem 418, 1541–1555 (2026). <https://doi.org/10.1007/s00216-026-06315-6>
- M. Horstmann, C. Derrick Quarles Jr, S. Happel et al.: Quantification of technetium-99 in wastewater by means of automated on-line extraction chromatography – anion-exchange chromatography – inductively coupled plasma-mass spectrometry. J. Anal. At. Spectrom., 2024, 39, 2774–2782, <https://doi.org/10.1039/D4JA00270A>
- Coma et al. "Automated separation of ⁹⁹Tc using plastic scintillation resin PSresin and overview automated modular separation system (OPENVIEW-AMSS)", Journal of Radioanalytical and Nuclear Chemistry (2019) 321:1057–1065. <https://doi.org/10.1007/s10967-019-06659-7>
- Barrera et al. "A new plastic scintillation resin for single-step separation, concentration and measurement of technetium-99" Analytica Chimica Acta 936 (2016) 259–266. <https://doi.org/10.1016/j.aca.2016.07.008>
- Tarancón et al. "A new plastic scintillation resin for single-step separation, concentration and measurement of ⁹⁹Tc", presented at the NRC9 (29/08/16 – 2/09/16, Helsinki, Finland)
- Hidex eBook "Liquid Scintillation Measuring Procedures: New Developments" <https://hidex.com/ebooks/liquid-scintillation-measuring-procedures/measuring-procedures/radionuclides-from-nuclear-fission-activities/2-3-14-tc-by-rad-disk-and-psresins/>
- J. Garcia & A. Tarancon, "Radionuclide determinations with PS Resin MASS WaterRadd", presented at the European Users Group Meeting in Cambridge (UK) - 21/09/2018, https://www.triskem-international.com/scripts/files/5bae2550c30ed4.50583030/11_j-garcia_a-tarancon_radionuclide-determinations-with-ps-resin_mass_waterradd.pdf
- Giménez et al. "A new method based on selective fluorescent polymers (PSresin) for the analysis of ⁹⁰Sr in presence of ²¹⁰Pb in environmental samples" Applied Radiation and Isotopes, 199, 2023, <https://doi.org/10.1016/j.apradiso.2023.110879>
- Tarancón et al. "Recent applications of Plastic Scintillation Resins », presented at Raddec-Triskem Workshop 2024, (18/04/24, Portsmouth, UK)
- Hyuncheol Kim and Gahyun Kim: "A simple and straightforward technique for analyzing radionuclides in sea water" presented at the Raddec/Triskem Workshop (as part of the 2024 LSC conference) on April 18 in Portsmouth (UK). Available online: https://www.triskem-international.com/scripts/files/662fa5023ef284.68970177/raddec_tki-ws-2024_6_kim.pdf
- Video KAERI Method: "Simple and Straightforward technique for Sr-90 analysis in seawater". Available online: <https://www.youtube.com/watch?v=i2R2b98oTCU>
- Hyuncheol Kim, Yoo Gyum Kang, Yong-Jin Lee et al.: "Automated extraction chromatographic radionuclide separation system for analysis of ⁹⁰Sr in sea water", Talanta 217 (2020) 121055. <https://doi.org/10.1016/j.talanta.2020.121055>
- Gahyun Kim, Sang-Do Choi, Jong-Myoung Lim, Hyuncheol Kim, Strontium-90 levels in seawater southeast of Jeju Island during 2021–2023, Marine Pollution Bulletin, Volume 193, 2023, 115258. <https://doi.org/10.1016/j.marpolbul.2023.115258>.

Products	Applications*
CL Resin	Pa, Ag, I removal
CU Resin	Cu from liquid Ni targets and from Zn targets
CU iSheets	Quality control of Cu labelled compounds
DGA Sheets	Quality control of Ra-223, Pb-212, Ac-225/Bi-213, Ge-68/Ga-68
ZR Resin	Zr, Ga, Ge, Ti
TK101 Resin	Ra, Pb
TK102 Resin	Ra/Ba separation, Pb, Sr
TK200 Resin	Ga-67/8, Zn, Actinides
TK201 Resin	Cu from solid Ni targets, Tc, Re
TK250 Resin	Cu separation from Ni and Zn targets
TK202 Resin	Tc from Mo targets, Re
TK221 Resin	Actinides, Lanthanide separation and purification (e.g. Lu-177), Ac-225 purification
TK222 Resin	Ac-225, Lu-177
TK225 Resin	Lanthanides removal from acidic effluents
TK227 Resin	Sr-90 determination via Y-90
TK211/2/3 Resins	Lanthanide separation (e.g. nca Lu-177, nca Tb-161)
TBP Resin	Sn, Zr, Sc, Ga, Actinides
TK400 Resin	Pa, Ga, Fe, Nb, Mo, Po
TK-GA Discs	Screening for actinides by alpha spectrometry, QC
Guard Resin	Removal of organic impurities, Ge-68
TK-TcScint	Tc-99 measurement via LSC
TK-SrScint	Separation and determination of Sr-89/90 via LSC

*Main applications are shown in blue

Upcoming new products :
TK300 Resin, SE Resin,
New iSheets and Discs,...



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